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# Fusion for Medical Radionuclides

24 September 2025





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# Welcome at OAS

**Paul Smith, Senior Business Development Manager  
MTC**



# What is Oxfordshire Advanced Skills (OAS)?



# Who are we



TBC



L2 Lean Operative



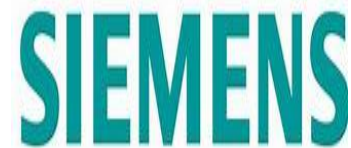
L3 Machining  
L3 Mechatronics  
L3 Tech Support  
HNC



L2 Eng Operative/ Lean  
L2 Nuclear Health  
Physics  
L2 Nuclear Operative  
L3 Machining  
L3 Tech Support  
L3 Mechatronics  
HNC  
L6 Nuclear Scientist and  
Nuclear Engineer (2026)

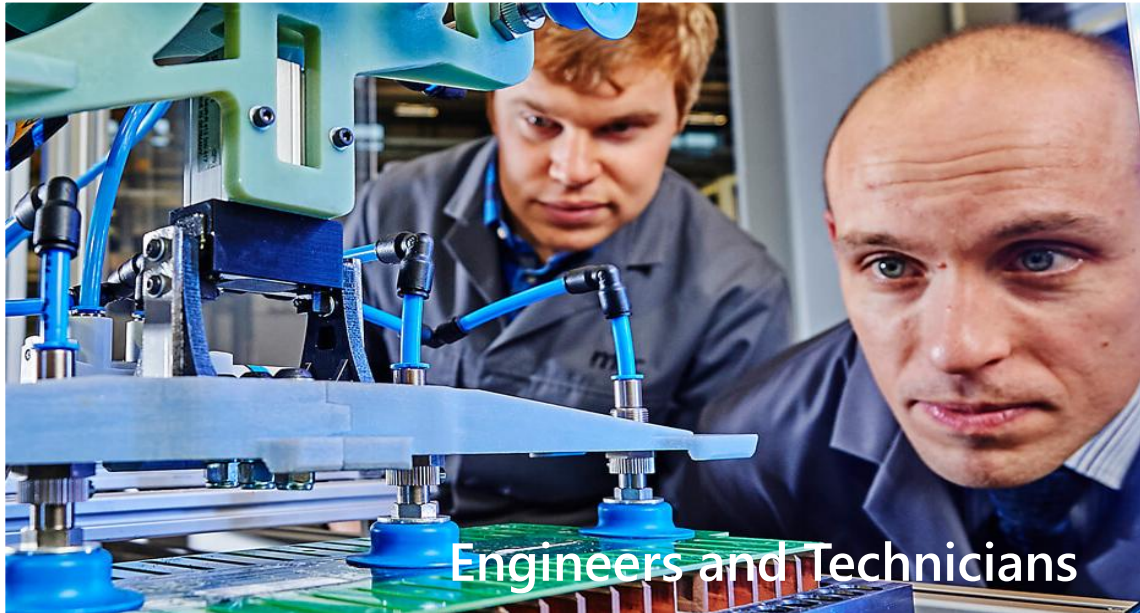


# Who we work with





# Who are our courses for?





# Welding

- MIG / MAG
- Competency checking
- Nuclear TIG welder
- NWIT
- Tig Level 3 modules
- Mechanised & orbital TiG
- L3 welding inspection course







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# Welcome from UKAEA

Heather Lewtas, Chief Development Officer  
UKAEA





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# Introduction

**Stefano Borini, Senior Innovation Lead**  
**UAKEA**



# Agenda

## 10:30 – 12:30: Presentations

Lee Evitts & Jessica Hollis, <b>UKAEA</b>	Exploration of Fusion technology for Medical Radionuclides at UKAEA
Jennifer Young, <b>Barts Cancer Institute - Queen Mary University of London</b>	Radionuclides for Health UK

## Coffee Break

Peter Ivanov, <b>National Physical Laboratory</b>	Development of Radiochemical Purification Methods for Emerging Medical Radionuclide Standards
Ram Mullur, <b>Astral Systems</b>	Plasma to Patients
Ross Radel, <b>SHINE</b>	Fusion Technologies for Industrial and Medical Applications

## 12:30 – 13:30: Networking Lunch

## 13:30 – 14:15: Panel Discussion

Jamie Townes (UKIFS)  
Talmon Firestone (Astral Systems)  
Marta Barrabino (Tokamak Energy)  
Jennifer Young (Barts Cancer Institute - Queen Mary University of London)  
Kathy Chan - (Institute of Cancer Research)

## 14:15 – 14:30 | Closing Remarks

## 14:30 – 15:30 | Networking & Poster Session

**UKAEA**

# Novel radionuclide production with high-energy D-T neutrons

Dr Lee J. Evitts ([lee.evitts@ukaea.uk](mailto:lee.evitts@ukaea.uk))

Senior Nuclear Physicist (Computational) | Team Leader, UKAEA



# Applied Radiation Technology (ART)

**Experimental**  
Kim Lennon

Gamma spectrometry on research measurements, waste assay (e.g. for decommissioning) and detector development (e.g. neutron diagnostics)

**Validation & Verification**  
Callum Grove

Experimental support (e.g. shielding irradiation), benchmarking codes incl. fluid activation

**Computational**  
Lee Evitts

Nuclear analysis (neutronics) to support design of projects at UKAEA (e.g. STEP, LIBRTI, JET) and external (e.g. ITER, Gauss Fusion, General Fusion)

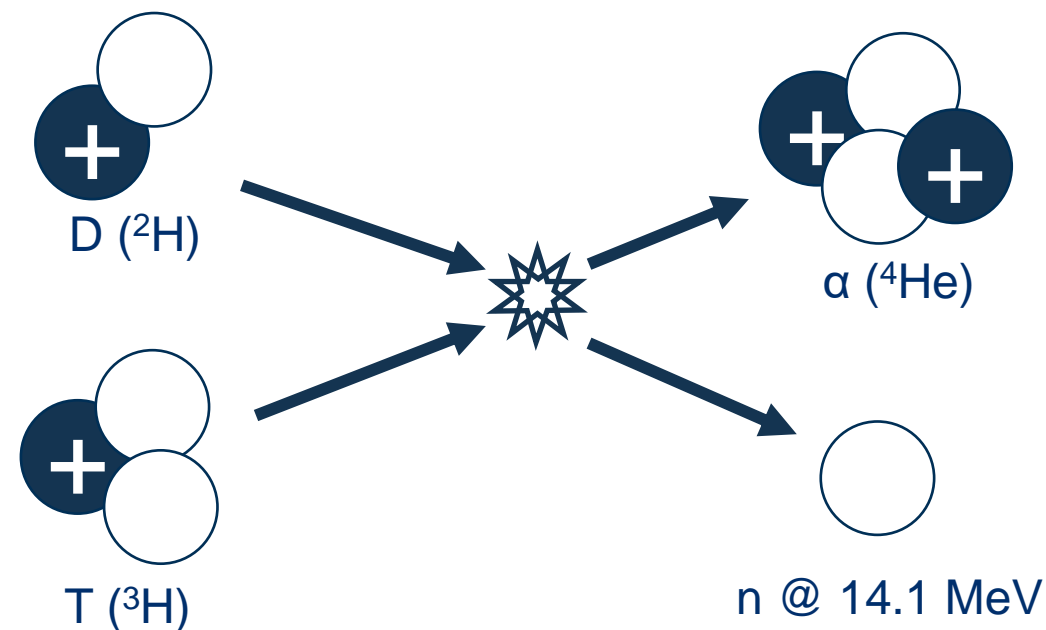
# Fusion

A nuclear reaction, where two (or more) nuclides can form one larger nuclide + emit energy

In the context of this talk, referring to D-T fusion, which emits a **14.1 MeV** neutron.

This high-energy neutron could be:

1. Multiplied/moderated to produce current nuclides (e.g.  $^{177}\text{Lu}$ )
2. Used as-is, to produce more novel nuclides or present alternate pathways






# Fusion vs fission neutrons

## Fission:

Majority of neutrons in a research reactor are likely thermalized ( $\sim 0.0025$  eV), aiming for (n, $\gamma$ ) reactions, where reaction cross-section is largest!


$^{91}\text{Nb}$ 680 y $\epsilon+\beta+=100\%$	$^{92}\text{Nb}$ $3.47\text{e}+7$ y $\epsilon+\beta+=100\%$	$^{93}\text{Nb}$ STABLE 100%	$^{94}\text{Nb}$ $2.04\text{e}+4$ y $\beta^-=100\%$
$^{90}\text{Zr}$ STABLE 51.45%	$^{91}\text{Zr}$ STABLE 11.22%	$^{92}\text{Zr}$ STABLE 17.15%	$^{93}\text{Zr}$ $1.61\text{e}+6$ y $\beta^-=100\%$
$^{89}\text{Y}$ STABLE 100%	$^{90}\text{Y}$ 64.046 h $\beta^-=100\%$	$^{91}\text{Y}$ 58.56 d $\beta^-=100\%$	$^{92}\text{Y}$ 3.54 h $\beta^-=100\%$


(n,  $\gamma$ ) 


## Fusion:


Higher energy opens up more available routes (more not shown)...

$^{91}\text{Nb}$ 680 y $\epsilon+\beta+=100\%$	$^{92}\text{Nb}$ $3.47\text{e}+7$ y $\epsilon+\beta+=100\%$	$^{93}\text{Nb}$ STABLE 100%	$^{94}\text{Nb}$ $2.04\text{e}+4$ y $\beta^-=100\%$
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(n,  $\alpha$ ) 

(n, p) 

(n, d) 

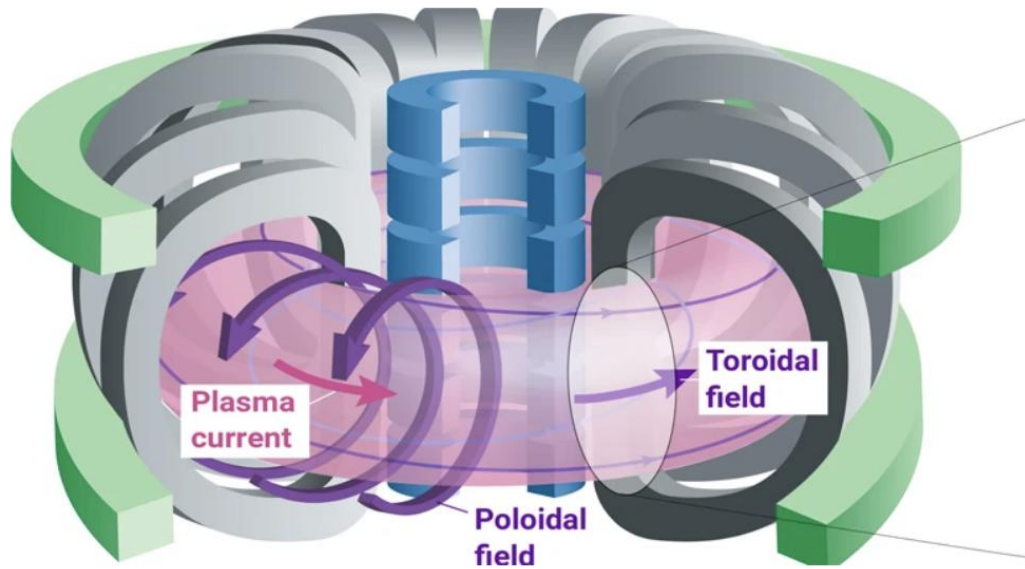
(n,  $\gamma$ ) 

Double-edged sword, more accessible nuclides but potential for more impurities!

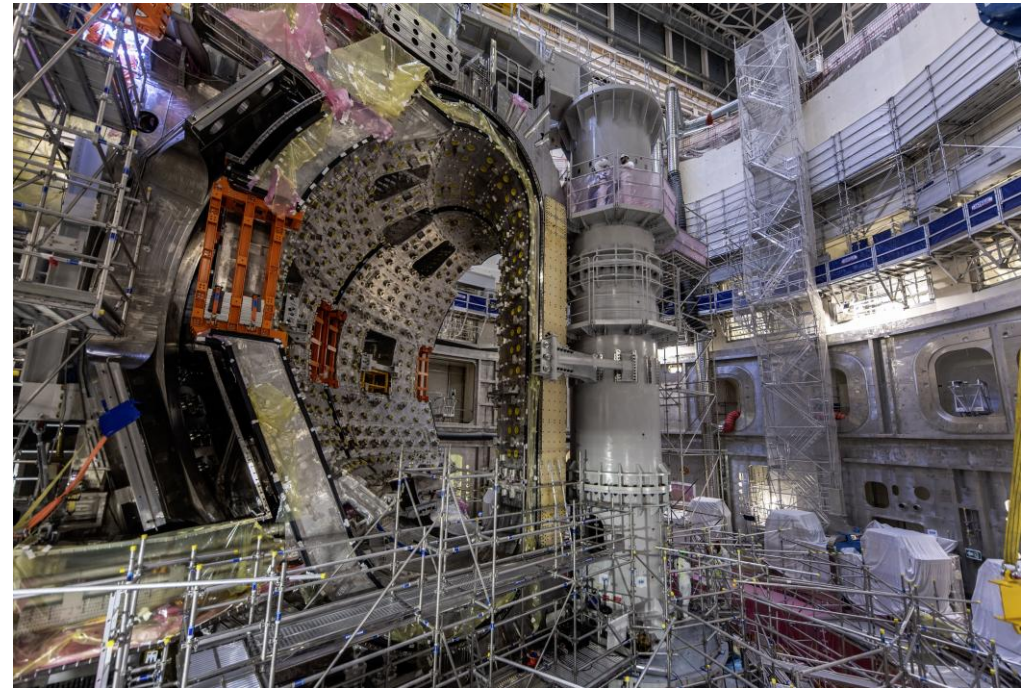
# Fusion machines

A whole range of active/proposed concepts including accelerators, magnetic or inertial confinement, magnetic mirrors, etc.

With magnetic confinement, strong magnetic fields contain a hot plasma, examples include **tokamaks** (e.g. ITER, STEP, MAST-U) and stellarators.

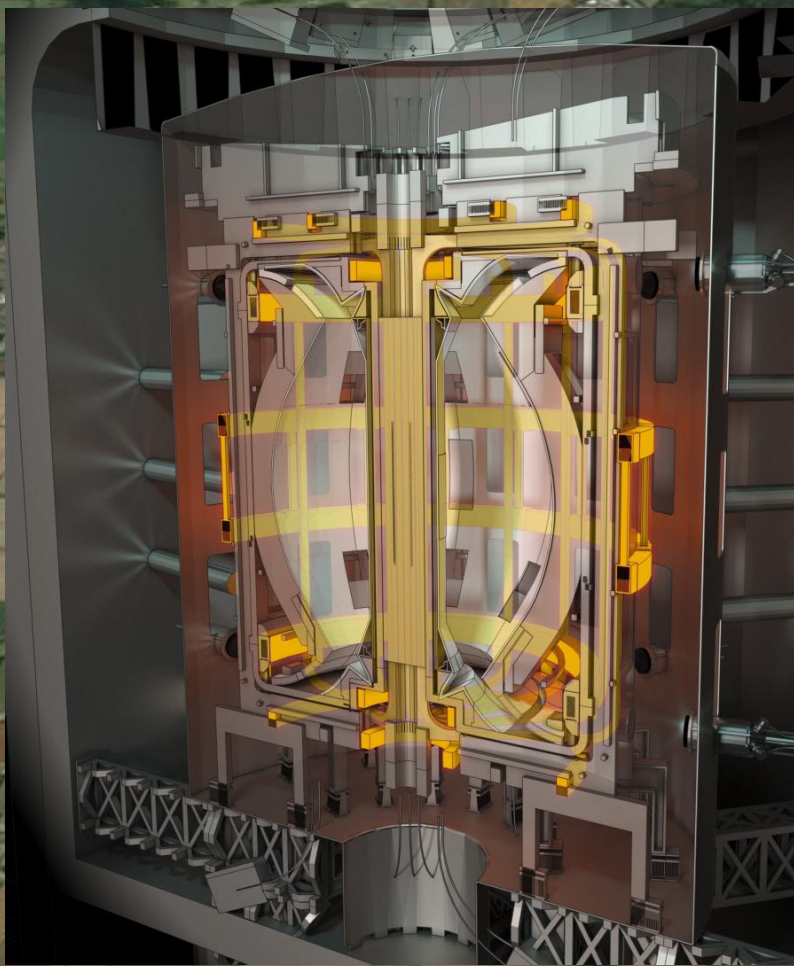


Tokamak Design ([REF](#))



ITER Construction ([REF](#))





**STEP**  
FUSION

DELIVER A UK PROTOTYPE FUSION ENERGY PLANT, TARGETING 2040, AND A  
PATH TO COMMERCIAL VIABILITY OF FUSION.

STEP MISSION



# Production feasibility study



Applied Radiation and Isotopes  
Volume 226, December 2025, 112163



## Theoretical novel medical isotope production with deuterium-tritium fusion technology

Lee J. Evitts <sup>a</sup>  , Philip W. Miller <sup>b</sup>, Chiara Da Pieve <sup>c 1</sup>, Andrew Turner <sup>a</sup>, Stefano Borini <sup>a</sup>

<sup>a</sup> UKAEA (United Kingdom Atomic Energy Authority), Culham Campus, Abingdon, Oxfordshire, OX14 3DB, UK

<sup>b</sup> Molecular Sciences Research Hub, White City Campus, Imperial College London, London, W12 0BZ, UK

<sup>c</sup> The Institute of Cancer Research, 123 Old Brompton Rd, London, SW7 3RP, UK

Recently accepted to Applied Radiation and Isotopes:  
<https://www.sciencedirect.com/science/article/pii/S0969804325005081>



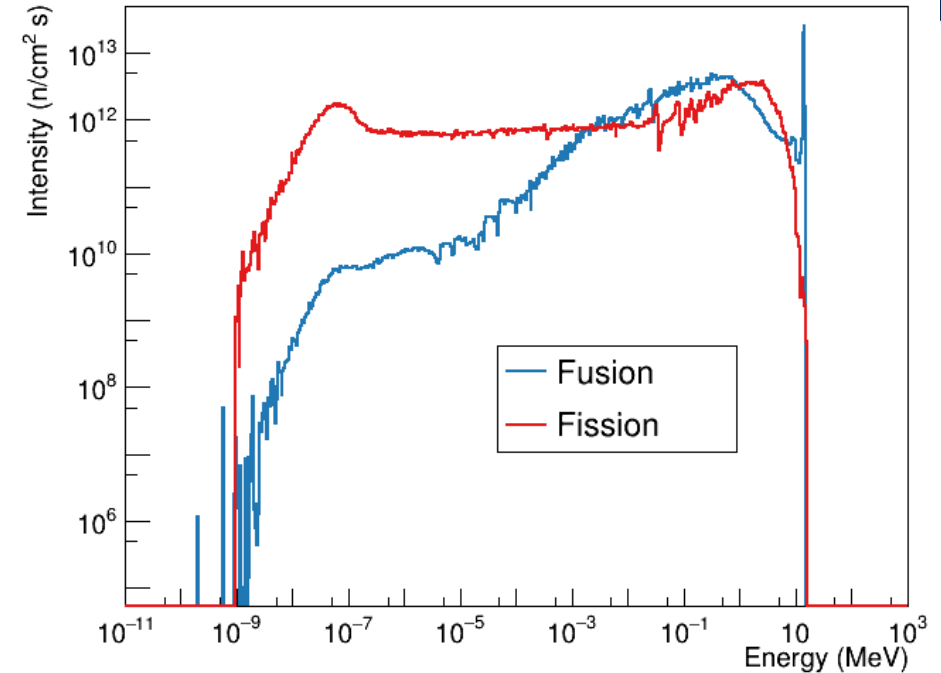
# Methodology & Assumptions

Identified all radionuclides that decay to a stable daughter with a relevant half-life (not just existing medical nuclides), and their feasible reactions.

Batch-wise inventory calculations performed with:

- FISPACT-II <sup>[1]</sup> inventory software,
- TENDL-2019 <sup>[2]</sup> nuclear data,
- example neutron spectrum in a tokamak-type device (right)<sup>[3]</sup>,
- Neutron flux of  $10^{14}$  n/(cm<sup>2</sup> s),
- 1 g of solid material

The results should be scalable to a device



Assumptions include:

- Target materials are 100% isotopically pure.
- Elements can be extracted with 100% efficiency, isotopes cannot.
- Irradiation period is three half-lives of the product.

[1] Sublet J. Ch. et. al. Nucl Data Sheets. (2017) 139

[2] Koning A.J. et. al. Nucl. Data Sheets. (2019) 155

[3] Fusion spectrum generated in this work with OpenMC & Paramak neutron transport calculations. Fission spectrum from HFR.

# Beta ( $\beta^-$ ) Emitters

The list of potential/novel beta emitters is extensive (right), highlights include:

<b>Copper-67</b>	$T_{1/2} \sim 62$ h Energy ( $\beta^-$ ) $\sim 100$ - $200$ keV Energy ( $\gamma$ ) $\sim 180$ keV
Existing Route	Cyclotron ( $^{68/70}\text{Zn}$ targets) but $^{64}\text{Cu}$ is also produced; potentially difficult to separate
Fusion Route	$^{67}\text{Zn}(n, p)$ Molar Activity $\sim 1800$ GBq/ $\mu\text{mol}$ Activity $\sim 20$ GBq per target gram Isotopic purity $\sim 100\%$
Notes	Usable in SPECT Can rely on existing studies Could use DOTA as suitable chelator Paired with $^{64}\text{Cu}$ for theragnostic

$^{24}\text{Na}$	$^{90}\text{Y}$	$^{31}\text{Si}$
$^{41}\text{Ar}$	$^{109}\text{Pd}$	$^{32}\text{P}$
$^{42}\text{K}$	$^{112}\text{Ag}$	$^{77}\text{As}$
$^{47}\text{Sc}$	$^{136}\text{Cs}$	$^{83}\text{Br}$
$^{48}\text{Sc}$	$^{139}\text{Ba}$	$^{105}\text{Rh}$
$^{56}\text{Mn}$	$^{140}\text{La}$	$^{111}\text{Ag}$
$^{65}\text{Ni}$	$^{150}\text{Pm}$	$^{131}\text{I}$
$^{67}\text{Cu}$	$^{156}\text{Eu}$	$^{149}\text{Pm}$
$^{72}\text{Ga}$	$^{159}\text{Gd}$	$^{161}\text{Tb}$
$^{76}\text{As}$	$^{172}\text{Tm}$	$^{199}\text{Au}$
$^{78}\text{As}$	$^{173}\text{Tm}$	
$^{82}\text{Br}$	$^{175}\text{Yb}$	
$^{86}\text{Rb}$		



# Beta ( $\beta^-$ ) Emitters

<b>Yttrium-90</b>	$T_{1/2} \sim 64 \text{ h}$ Energy ( $\beta^-$ ) $\sim 900 \text{ keV}$ Energy ( $\gamma$ ) - none
Existing Route	Neutron ( $^{89}\text{Y}$ ) to produce carrier-added form Or slow decay from $^{90}\text{Sr}$ (U fission product)
Fusion Route	$^{93}\text{Nb}(n, \alpha)$ Molar Activity $\sim 1000 \text{ GBq}/\mu\text{mol}$ Activity $\sim 5 \text{ GBq}$ per target gram Isotopic purity $\sim 99.7\%$
Notes	Has already had extensive use/research behind it

$^{24}\text{Na}$	$^{90}\text{Y}$	$^{31}\text{Si}$
$^{41}\text{Ar}$	$^{109}\text{Pd}$	$^{32}\text{P}$
$^{42}\text{K}$	$^{112}\text{Ag}$	$^{77}\text{As}$
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$^{78}\text{As}$	$^{173}\text{Tm}$	
$^{82}\text{Br}$	$^{175}\text{Yb}$	
$^{86}\text{Rb}$		

# Beta ( $\beta^-$ ) Emitters

<b>Scandium-47</b>	$T_{1/2} \sim 3.3 \text{ d}$ Energy ( $\beta^-$ ) $\sim 500 \text{ keV}$ Energy ( $\gamma$ ) $\sim 160 \text{ keV}$
Existing Route	
Fusion Route	$^{50}\text{V}(n,\alpha)$ or $^{48}\text{Ca}(n, 2n)$ but low nat. abundance Molar Activity $\sim 1400 \text{ GBq}/\mu\text{mol}$ Activity $\sim 600 \text{ GBq}$ per target gram Isotopic purity $\sim 100\%$
Notes	Usable in SPECT Can rely on existing studies Could use DOTA as suitable chelator Paired with $^{44}\text{Sc}$ for theragnostic

$^{24}\text{Na}$	$^{90}\text{Y}$	$^{31}\text{Si}$
$^{41}\text{Ar}$	$^{109}\text{Pd}$	$^{32}\text{P}$
$^{42}\text{K}$	$^{112}\text{Ag}$	$^{77}\text{As}$
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$^{78}\text{As}$	$^{173}\text{Tm}$	
$^{82}\text{Br}$	$^{175}\text{Yb}$	
$^{86}\text{Rb}$		



# Auger & Alpha Emitters

Due to the available (n, 2n) reaction, some potential Auger emitters can also be created including:

$^{64}\text{Cu}$ ,  $^{77}\text{Br}$ ,  $^{89}\text{Zr}$ ,  $^{119}\text{Sb}$ ,  $^{135}\text{La}$  and  $^{201}\text{Tl}$

Their production yields wouldn't compete with a cyclotron, but fusion could offer a useful alternative

There is also a good feasibility to produce alpha emitters like  $^{212}\text{Bi}$  and  $^{225}\text{Ac}$ , see next talk(s)

# Conclusions / Future Work

It is theoretically viable to produce existing and novel medical isotopes with D-T fusion technology. However, a significant number of assumptions form the basis of this study which would need to be investigated further for any particular nuclide of interest.

For each nuclide of interest, further research would be required in, for example:

- Nuclear physics, measuring the reaction cross-sections
- Cost analysis, including acquiring and recycling target materials.
- Target design to optimize production quantities and purities.
- Development of target and product purification/extraction techniques.
- Development of appropriate GMP practices.
- Related clinical research e.g., attaching to target vectors, in-vivo stability, uptake, retention, dose, etc.



# Thank you

# Medical Isotope Production Modelling

Jessica Hollis, David Foster and Mark Gilbert

UKAEA



# Talk outline

UKAEA engaged with NIRO to work on four packages aimed to help understand the application of *fusion* neutrons to medical isotope production.

The work packages were:

**Task 1:** The comparison of Mo99 production from Uranium of varying enrichments within a fission and fusion spectrum.

**Task 2:** The comparison of Mo99 production from Uranium of varying enrichments within a fusion power plant spectrum – varying locations within a breeder blanket.

**Task3:** A scoping study for the potential production routes of various medical isotopes of interest.

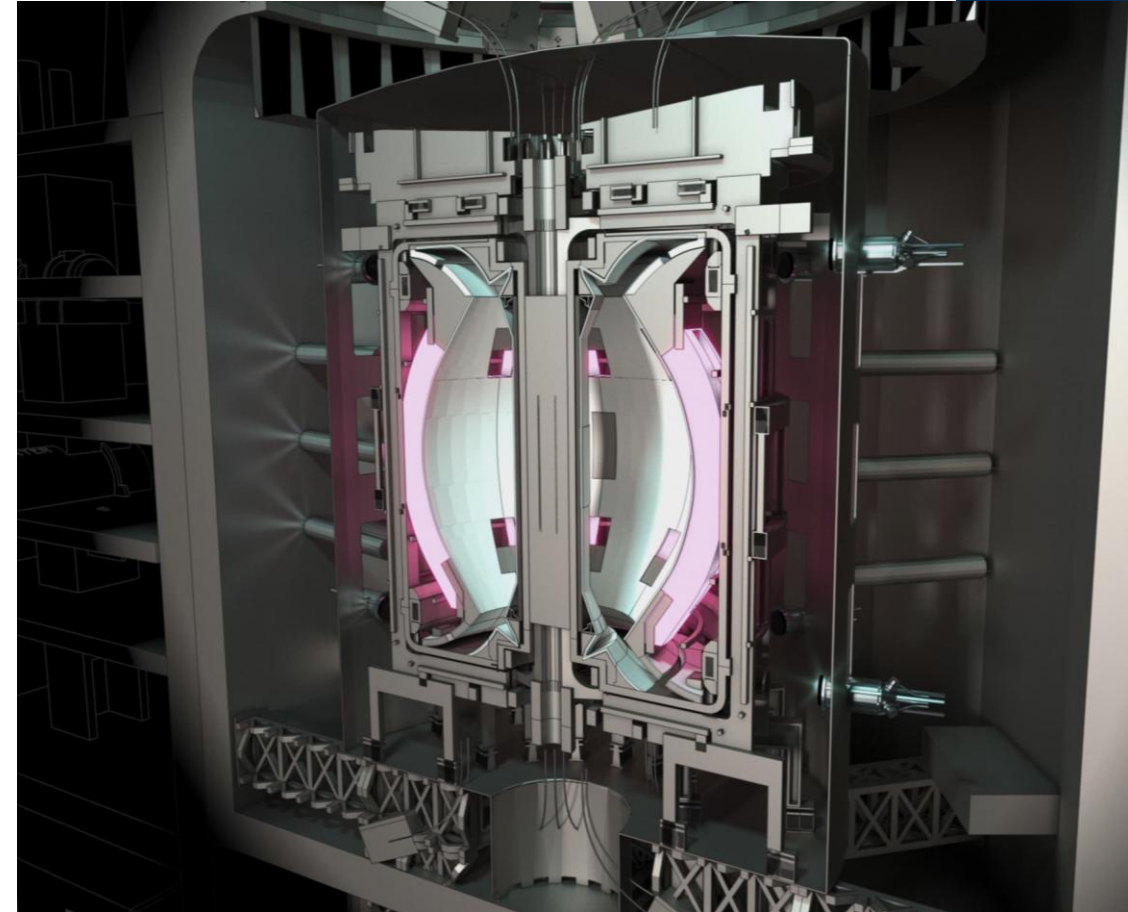
**Task 4:** A conceptual study (based on the results from the previous reports) to add a target station focused on radionuclide production, to the proposed LIBRTI test facility.

# UKAEA activation calculations

For the delivery of fusion power UKAEA needs to understand the activation (transmutation) of reactor components

- Waste classification (£Billion sector)
- Waste mitigation – material engineering (£Billion sector)
- Worker dose – decay gammas

**Solution:** FISPACT-II – UKAEA's nuclear inventory code



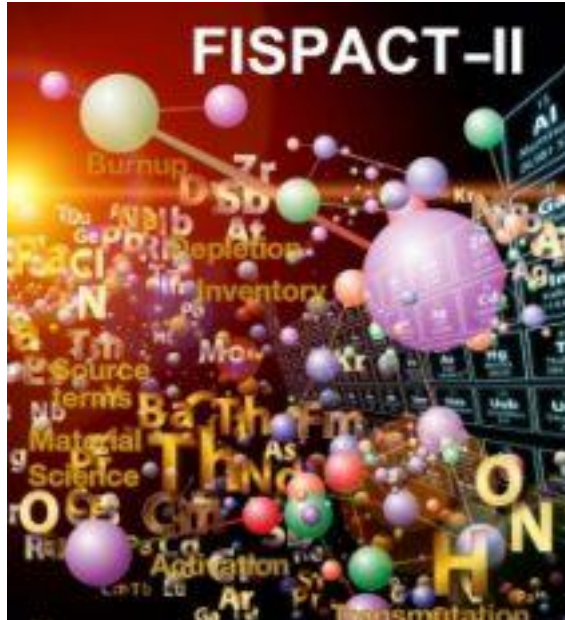
*A STEP type blanket*

<https://ccfe.ukaea.uk/uk-to-launch-search-for-industry-partners-to-develop-prototype-fusion-energy-plant/step-tokamak-breeder-blankets-ccfe/>

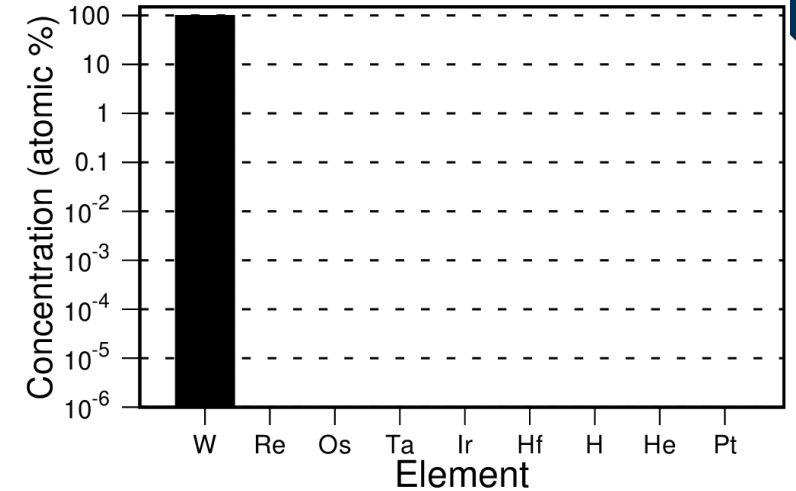
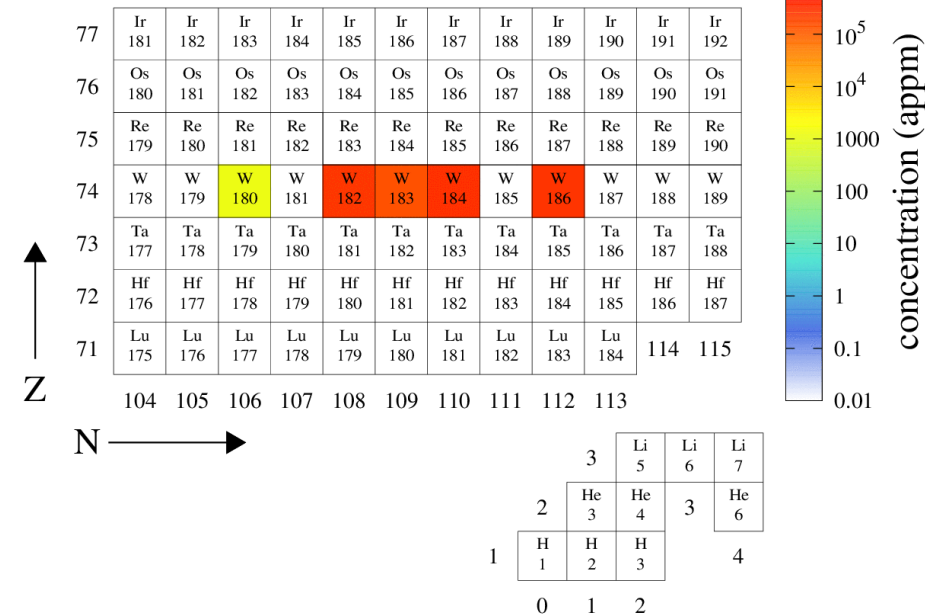


# FISPACT-II - UKAEA nuclear inventory code

Models the transmutation of a material



Time: 0.00 seconds



*Tungsten transmutation under a fusion spectrum (M. Gilbert).*

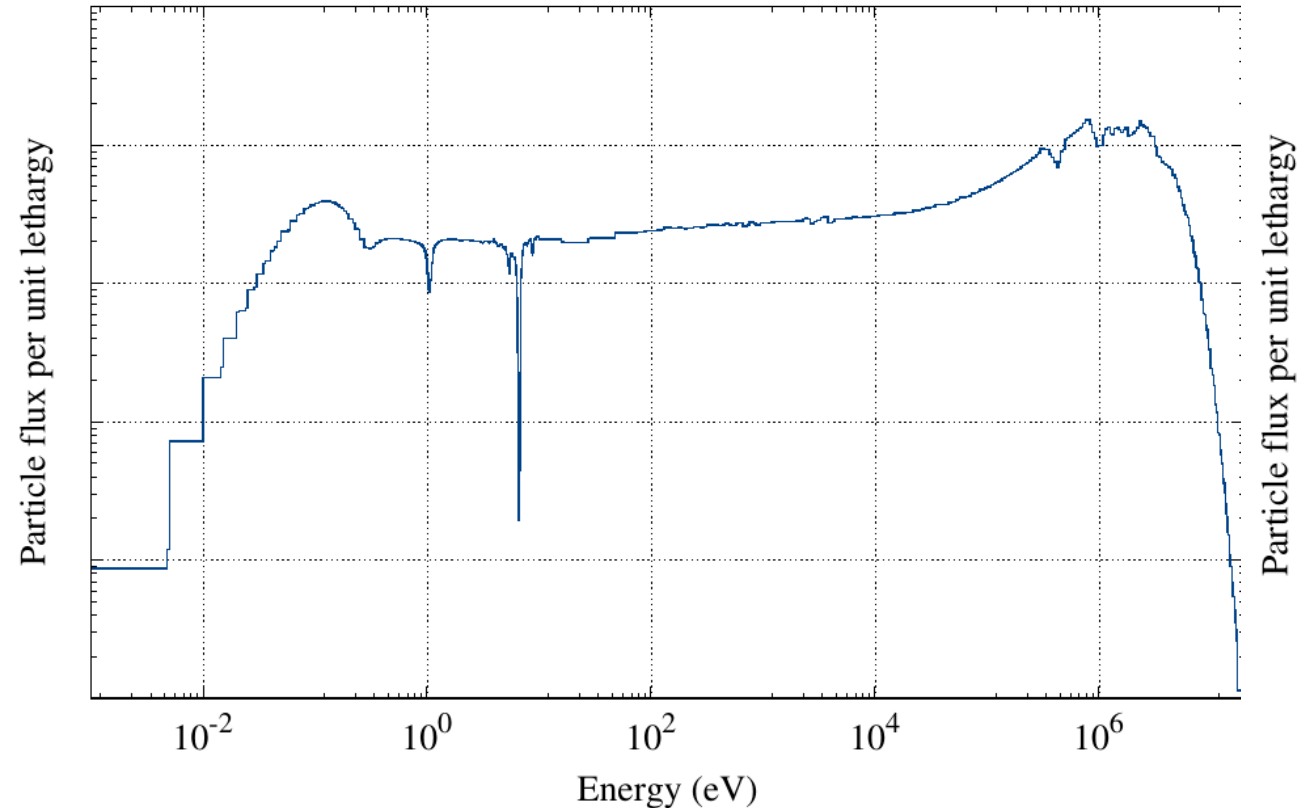
$$\frac{d}{dt}N_i = -N_i\lambda_i - N_i \sum_s \int_0^\infty \sigma_{is}(E)\phi(E)dE + \sum_{k \neq i} N_k \left( \lambda_{ki} + \int_0^\infty \sigma_{ki}(E)\phi(E)dE \right)$$

$N$  is a list of the number of nuclides,  $\phi$  is the projectile spectrum,  $\sigma$  is the cross sections and  $\lambda$  is the decay constant

# Task 1

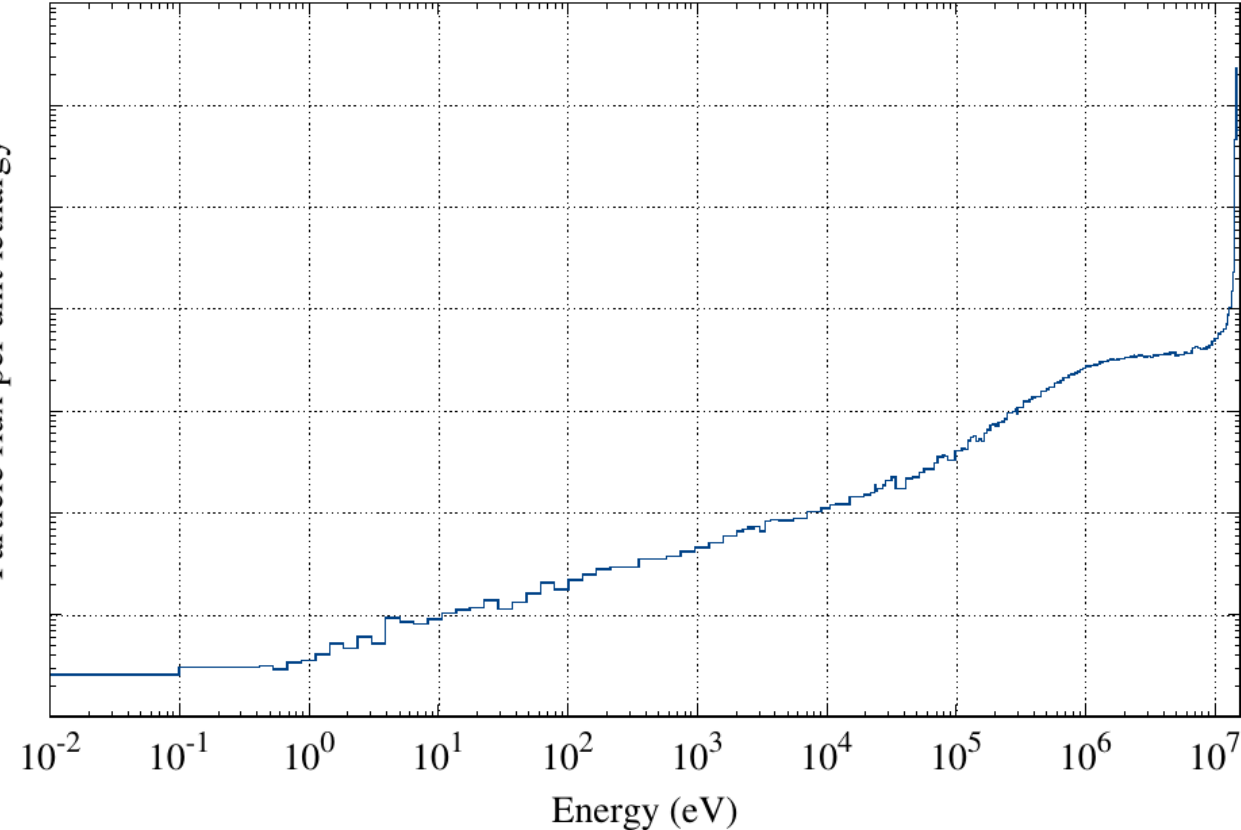
## Comparison of Mo99 production from Uranium of varying enrichments within a fission and fusion spectrum.

PWR-UO2-15 (1102 grps)



*Fission energy spectrum for a pressurised water reactor (PWR) at 15 GWd/THM. See FISPACT-II reference input spectra*

JAEA-FNS-pos3 (175 grps)



*Spectra obtained from the Fusion Neutron Source (FNS) experimental set-up at JAEA. See FISPACT-II reference input spectra*



# Task 1

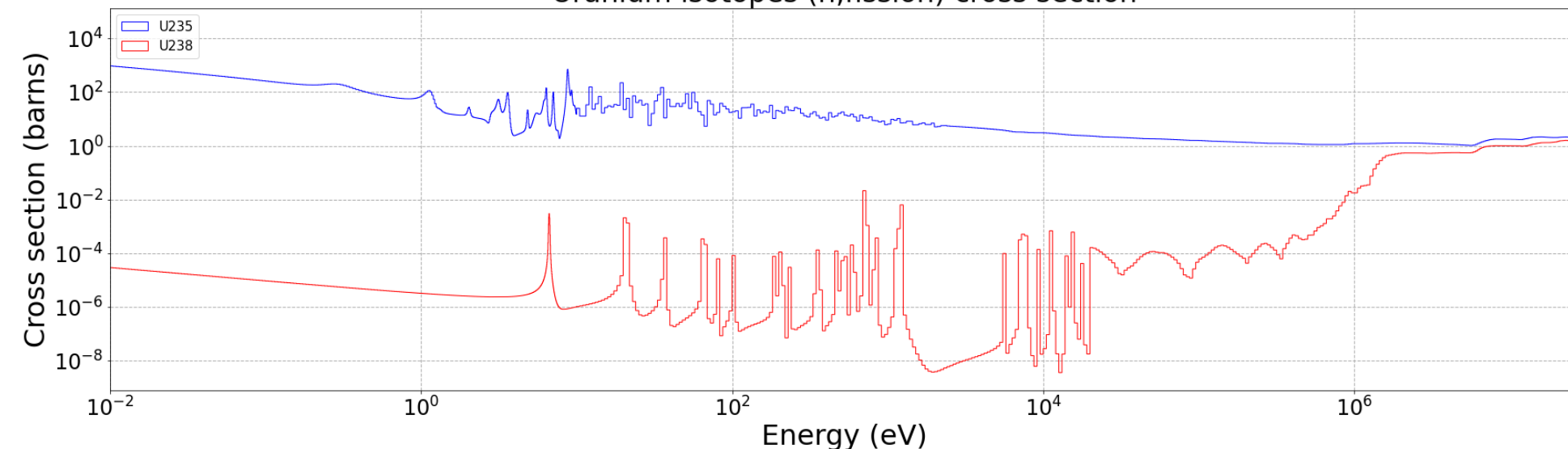
Mo99 yield (Bq) immediately post 7-day irradiation

Spectrum	1% U235 enrichment	5%U235 enrichment	10%U235 enrichment	19.7%U235 enrichment
Fission	6.919E+13	2.687E+14	5.181E+14	1.002E+15
Fusion - 14MeV monoenergetic	4.116E+12	4.215E+12	4.339E+12	4.580E+12
Fusion – FNS	3.843E+12	3.944E+12	4.069E+12	4.312E+12

- Mo99 yield is consistently at least an order of magnitude higher for the fission spectrum in comparison to the fusion spectra (higher flux and higher proportion of thermal neutrons)

- The fusion spectra shifts to energies where the cross section for U238 fission considerably increases (yet still below that of U235). The impact of enrichment is limited in these cases.

Uranium isotopes (n,fission) cross section



- May be optimal to use depleted uranium and not enriched uranium as it has lower cost and less regulation.

- Further research required to investigate the effect of neutron multiplicity for U238 fission.

# FISPACT-II can identify the pathway of how a nuclide is produced.

Enrichment

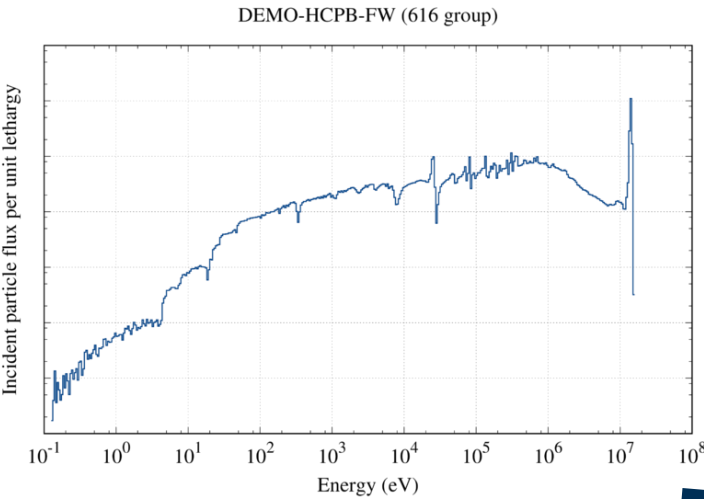
Pathway	Fission Spectrum (1% U enrichment)	Fission Spectrum (19.7% U enrichment)	Fusion Spectrum (1% U enrichment)	Fusion Spectrum (19.7% U enrichment)
U-235(n,F) → Zr-99 → Nb-99 → Mo-99	27.0	36.7	0.5	9.9
U-235(n,F) → Zr-99 → Nb-99m → Mo-99	15.4	20.9	—	5.6
U-235(n,F) → Y-99 → Zr-99 → Nb-99 → Mo-99	13.4	18.2	—	5.0
U-235(n,F) → Y-99 → Zr-99 → Nb-99m → Mo-99	7.6	10.4	—	2.9
U-235(n,F) → Nb-99 → Mo-99	4.8	6.5	—	3.8
U-235(n,F) → Sr-99 → Y-99 → Zr-99 → Nb-99 → Mo-99	1.4	1.9	—	—
U-235(n,F) → Nb-99m → Mo-99	0.9	1.3	—	0.6
U-235(n,F) → Sr-99 → Y-99 → Zr-99 → Nb-99m → Mo-99	0.8	1.1	—	—
U-235(n,F) → Mo-99 (direct)	—	0.5	—	—
U-238(n,F) → Y-99 → Zr-99 → Nb-99 → Mo-99	8.4	—	29.2	21.1
U-238(n,F) → Zr-99 → Nb-99 → Mo-99	6.3	—	24.7	17.9
U-238(n,F) → Y-99 → Zr-99 → Nb-99m → Mo-99	4.8	—	16.6	12.0

Mo production shifts from U235 to U238 in fusion due to the faster neutrons

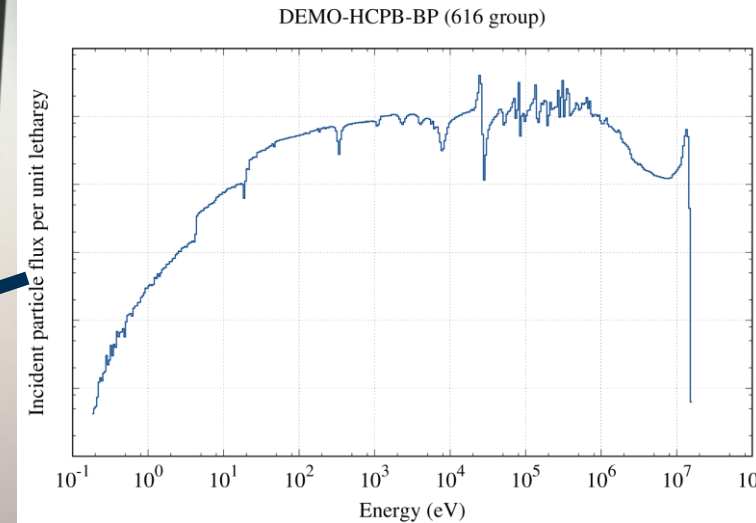
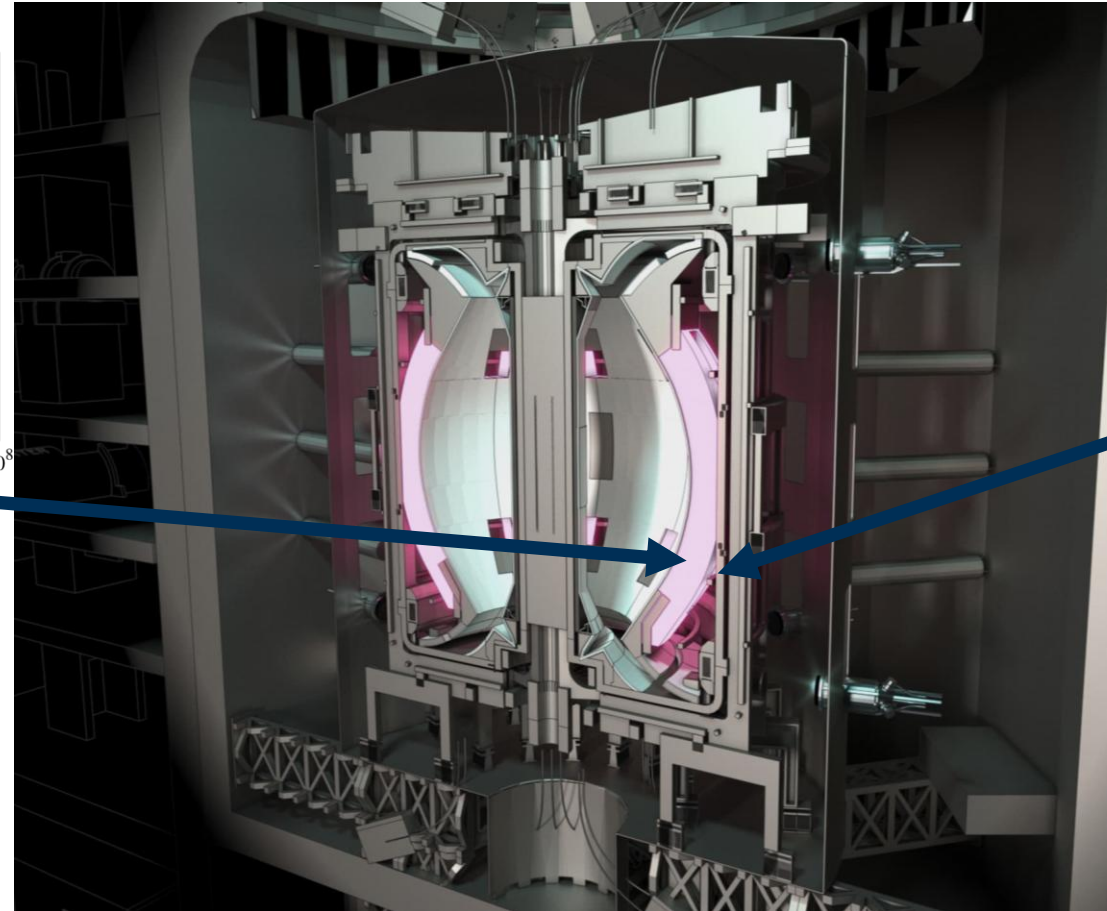


# Task 2

**The comparison of Mo99 production from Uranium of varying enrichments within a fusion power plant spectrum – varying locations within a breeder blanket.**



*DEMO fusion concept He-cooled pebble bed, first wall. FISPACT-II reference spectra*



*DEMO fusion concept He-cooled pebble bed, backplate. FISPACT-II reference spectra*

*A STEP type blanket*

<https://ccfe.ukaea.uk/uk-to-launch-search-for-industry-partners-to-develop-prototype-fusion-energy-plant/step-tokamak-breeder-blankets-ccfe/>

# Task 2

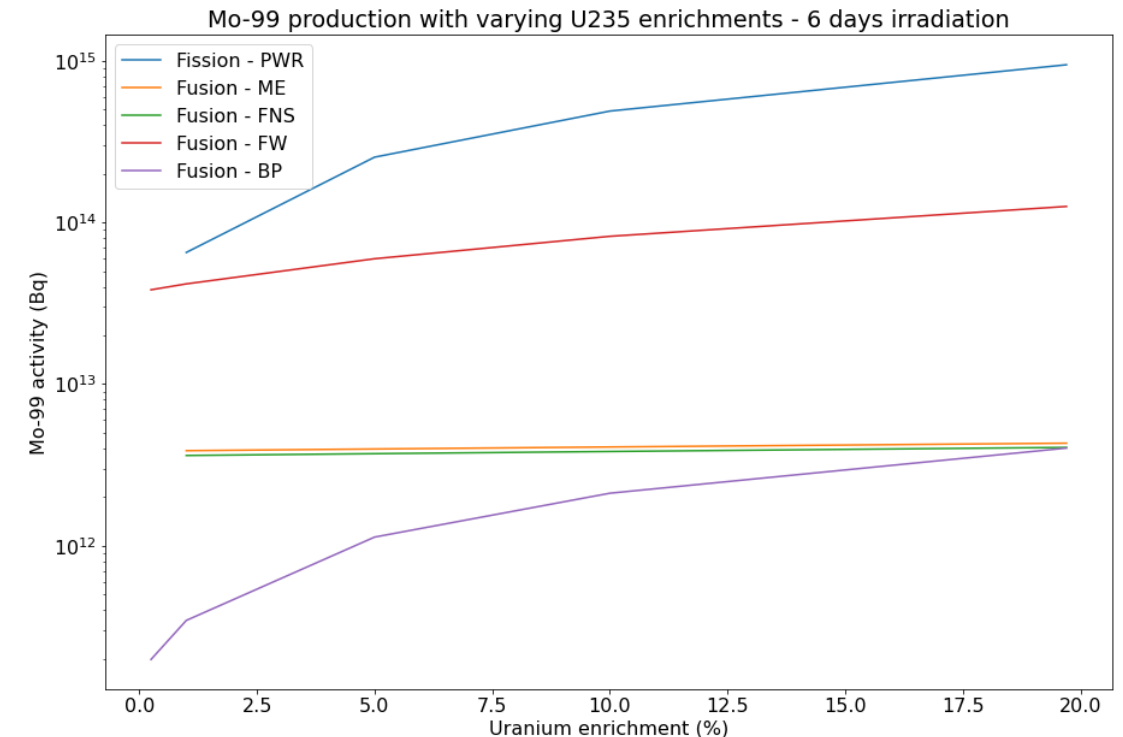
- Production of Mo99 from Uranium of varying enrichments within a fusion power plant spectrum (EU-DEMO D-T fusion reactor He-cooled pebble bed first wall and backplate).

Mo99 yield (Bq) immediately post 6-day irradiation

	0.25%	1%	5%	10%	19.7%
Firstwall	3.829E+13	4.165E+13	5.959E+13	8.202E+13	1.255E+14
Backplate	1.979E+11	3.452E+11	1.131E+12	2.113E+12	4.018E+12

- Mo99 yield is highest for the firstwall compared to the backplate due to the higher flux
- The yield increase between 0.25% and 19.7% U235 is less than an order of magnitude for the firstwall.

- As expected, the Mo99 yield for the PWR fission spectrum increases significantly with U235 enrichment
- Due to the down scatter of the ~14MeV neutrons, the backplate spectra is more favourable towards U235 fission (but has lower flux).
- Impact of enrichment is limited for the other fusion spectra (higher proportion of higher energy neutrons).



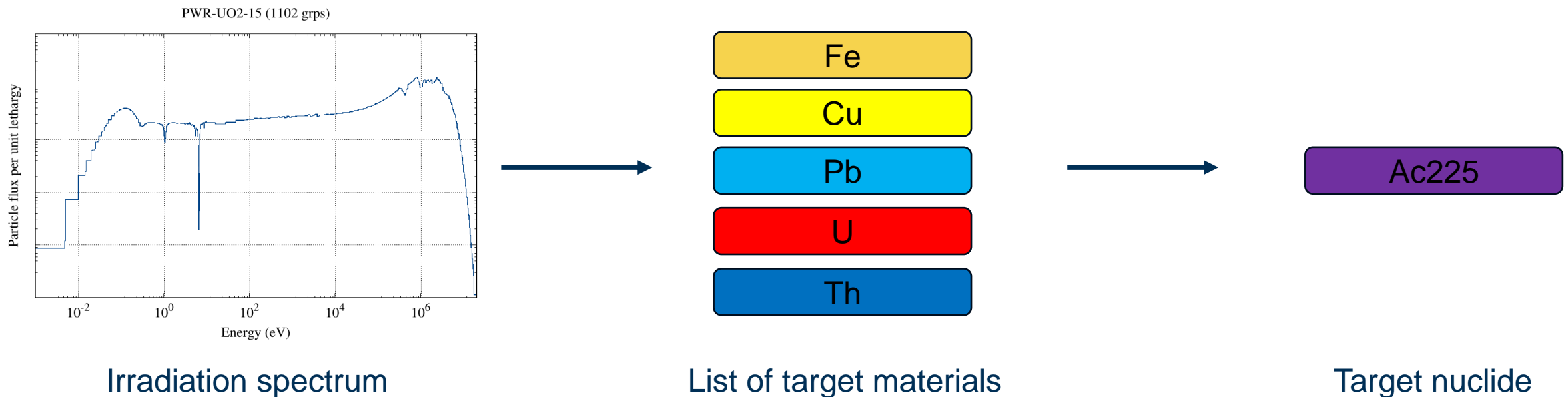
# Task 3

Investigation of the production possibilities for (Cu64, Ga68, Y90, Mo99, Tb149, Lu177, Pb212, Bi213, At211, Ra226, Ac225, Th227) from all target elements or nuclides

Aim:

- identify the targets which produce the highest yields of the medical isotope
- simulate both a fission and fusion neutron environment for comparison
- Identify production methods which do not use uranium – lower risks

Method

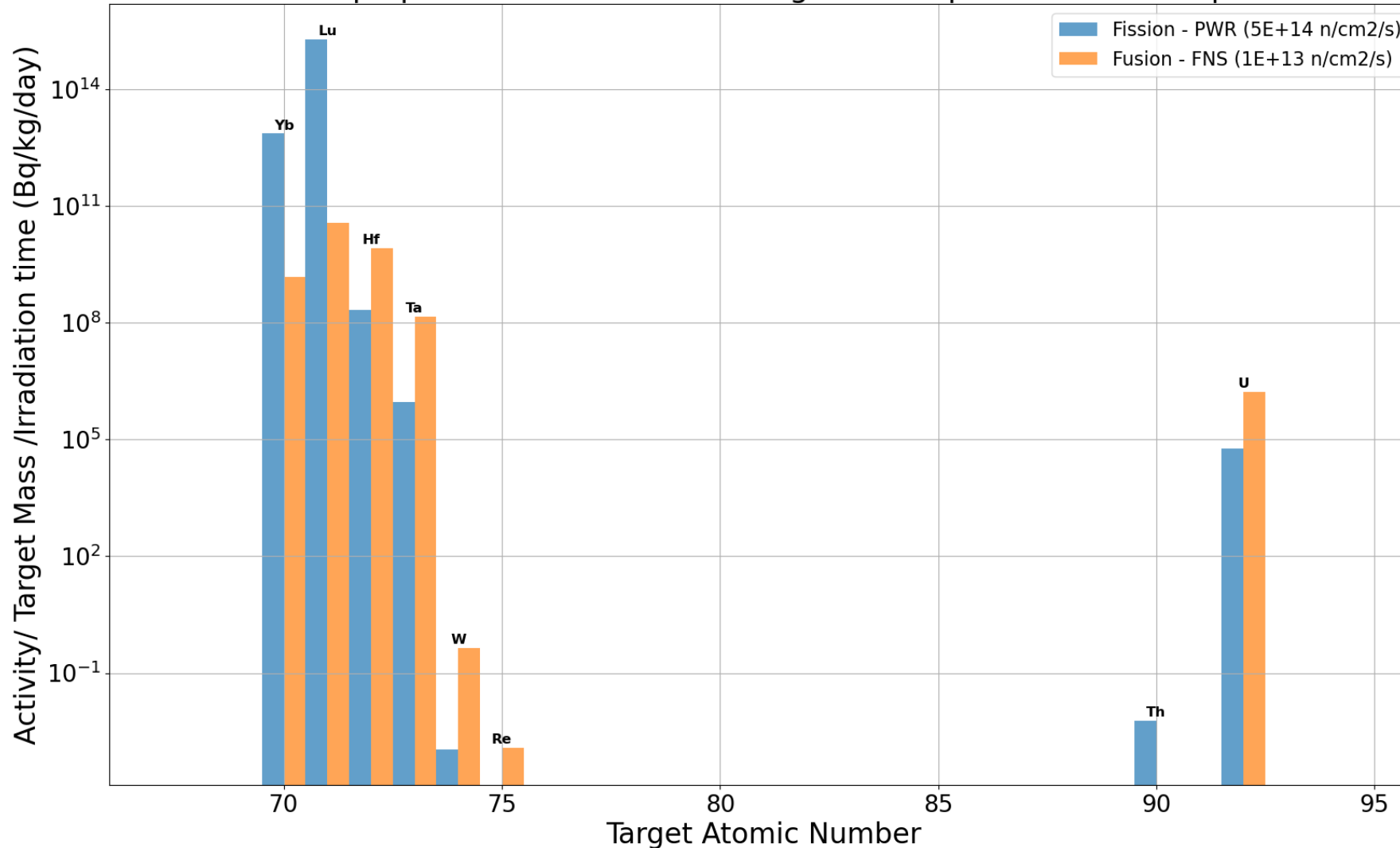




# Task 3:

## Lu177 production example

Medical isotope production for various targets and spectra for Lu177 production



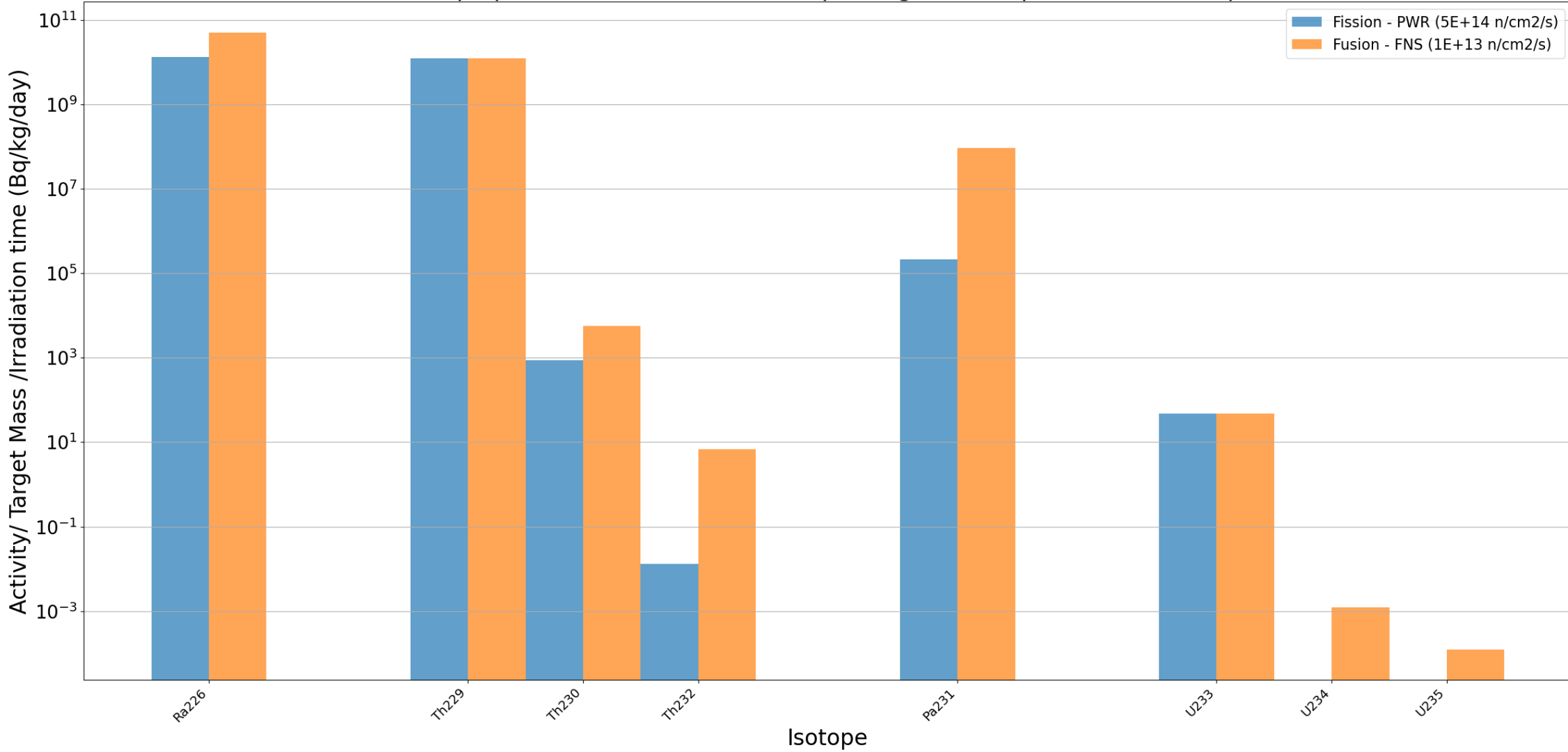
- Typical picture: islands of production around the atomic number of isotope of interest and around actinides (due to fission)
- E.g. max production of Lu177 from Lu as a target

Lu177 yield (activity/unit target mass/unit irradiation time – Bq/kg/day) obtained from the irradiation of naturally abundant targets with different neutron spectra.

# Task 3:

## Ac225 production example by nuclide.

Medical isotope production for various isotopic targets and spectra for Ac225 production

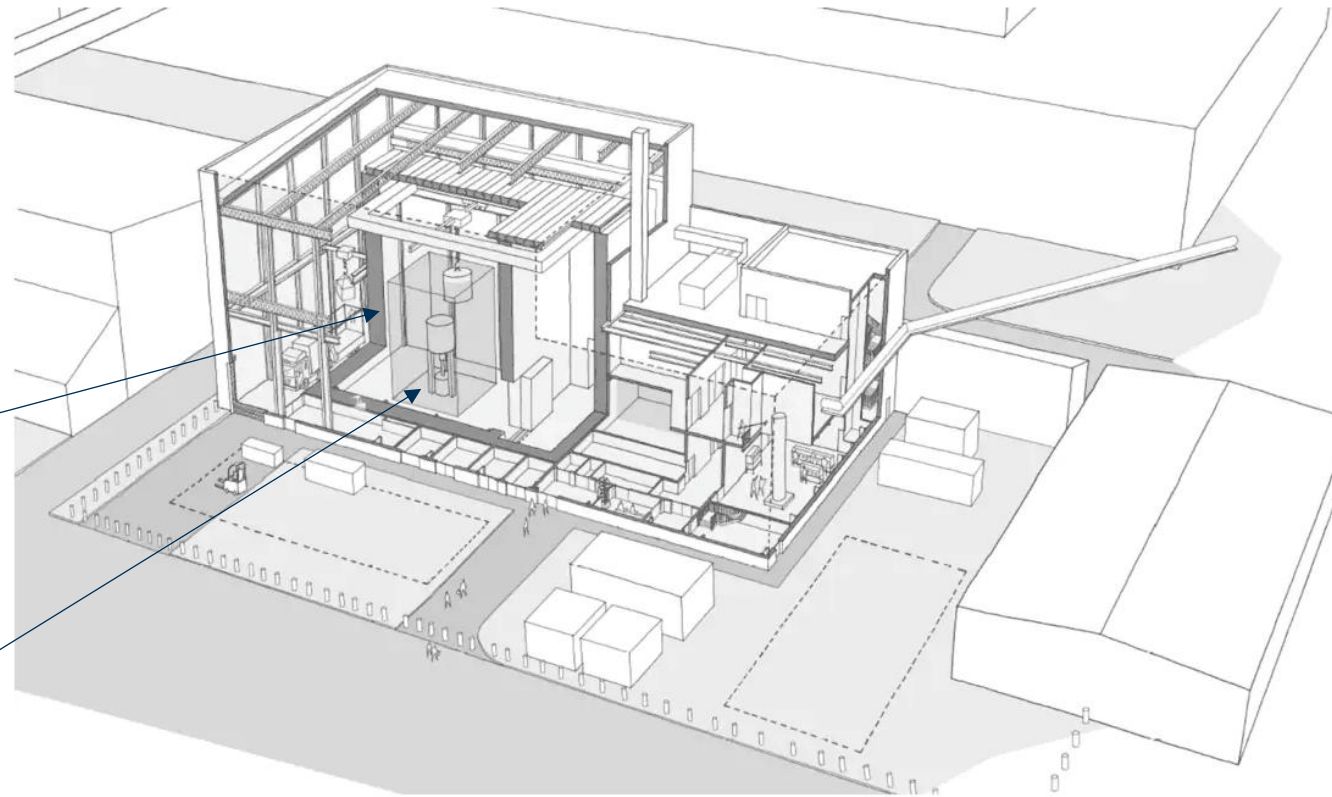


# Task 4: Small scale medical isotope production in the LIBRTI facility

- £200m Lithium Breeding Tritium Innovation (LIBRTI) program
- LIBRTI will develop a digital framework to model fusion blankets, develop new fusion blanket designs and drive the handling and development of tritium.
- Test tritium production of meter scale components
- Waste route
- Could be used to produce small amounts of medical isotopes in the future

Block house

Neutron source

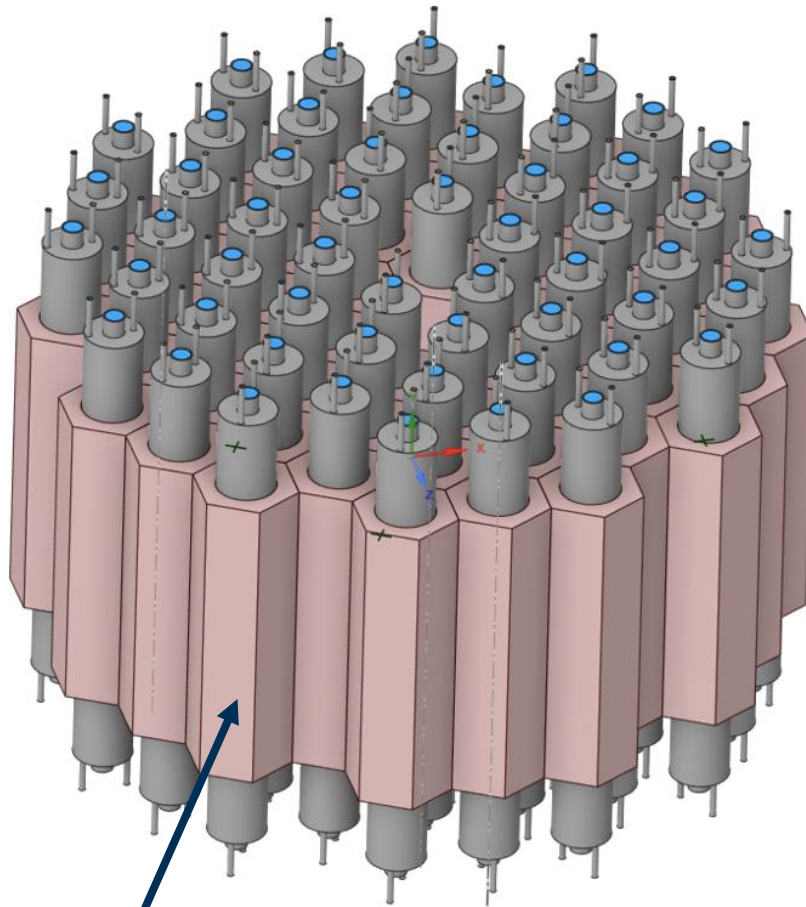


Concept sketch of the LIBRTI facility. Note the central grey box which represents the LIBRTI block house.  
(<https://ccfe.ukaea.uk/programmes/fusion-futures/librti/>)

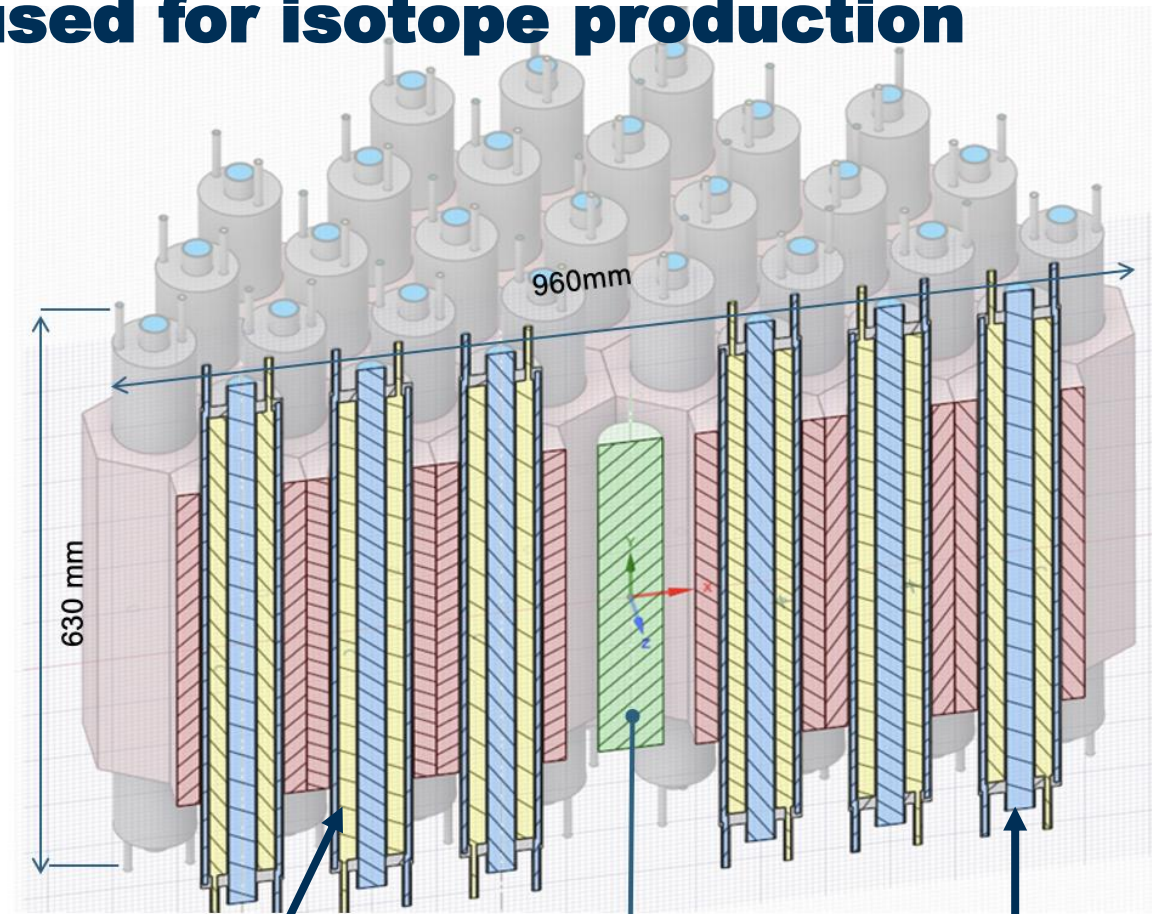


# LIBRTI modular pincell

A breeder blanket concept used for isotope production



Neutron multiplier

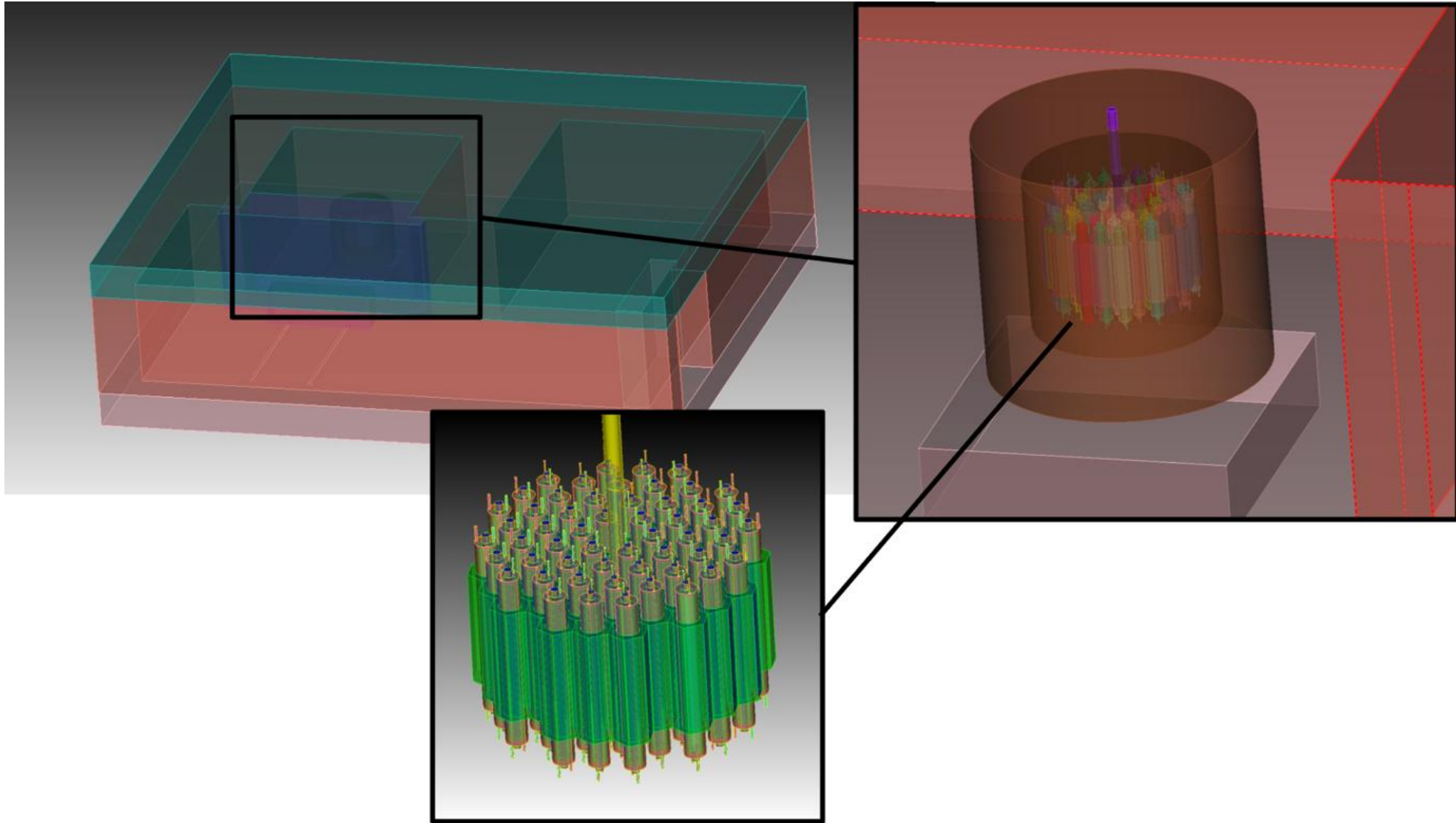


14MeV neutron source

Isotope breeder material

Coolant

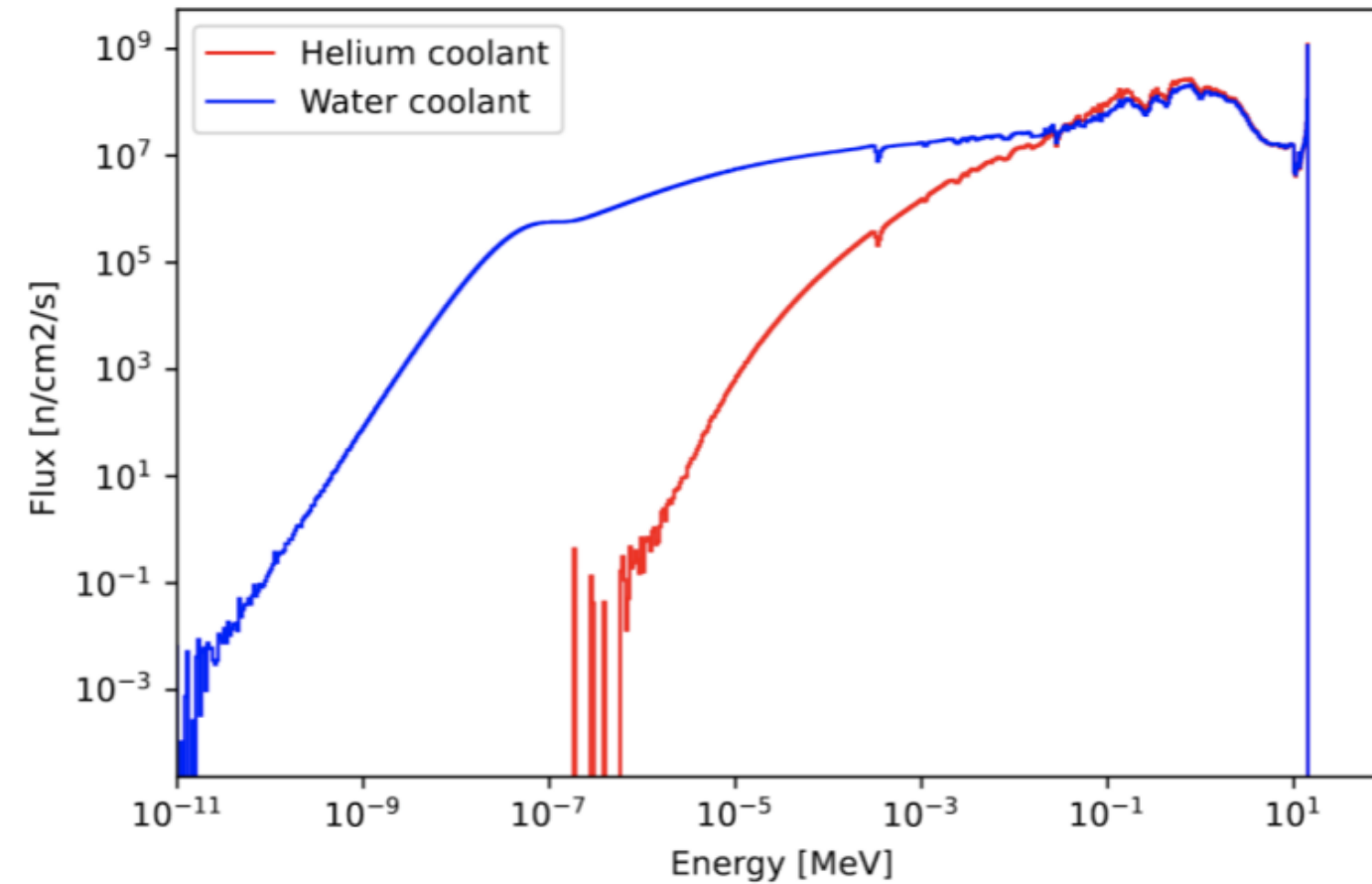
# Neutron spectrum modeling



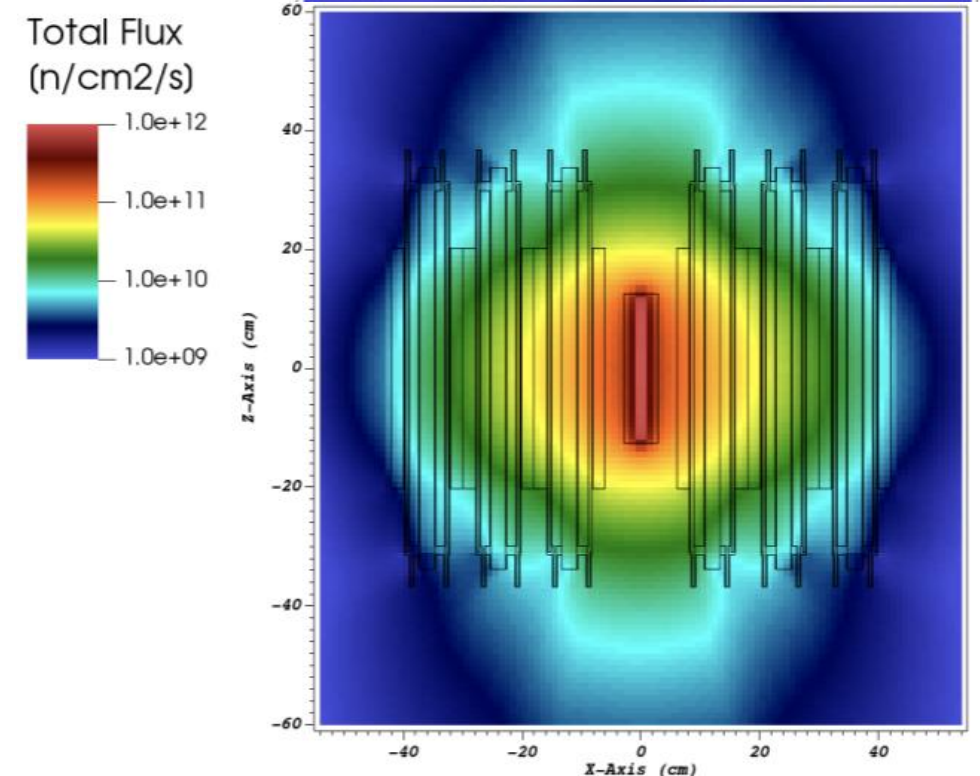
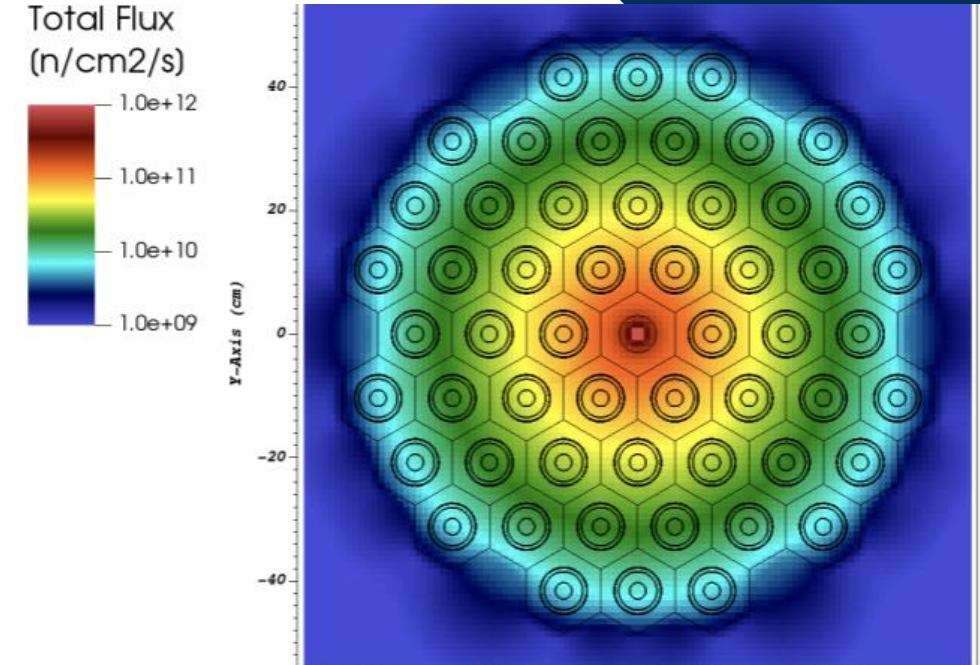
Neutron transport (OpenMC) model of the Modular pincell within the LIBRTI facility



# Neutron spectrum modeling



Comparison of the average neutron spectrum within the pincell for different coolants. This shows that the neutron spectrum can be engineered, which is important to maximise key medical isotope production.





# Isotope production within the LIBRTI facility

		Yield (Bq/kg/day) in LIBRTI flux environments		
Target	Medical Isotope	Minimum	Average	Maximum
Mo100	Mo99	2.725E+8	3.065E+9	1.666E+10
Cu	Cu64	8.945E+8	4.855E+9	2.009E+10
Zn	Cu64	1.577E+8	1.342E+9	6.648E+9
Lu	Lu177	2.791E+7	8.601E+7	1.916E+8
Ra226	Pb212	1.722E+7	2.231E+8	1.250E+9
Ra226	Ac225	1.187E+6	1.206E+7	6.380E+7

*Medical isotope yield (activity per target mass per irradiation time – Bq/kg/day) from an identified target irradiated with the expected spectra at different reference points within the model.*

# Summary

- Fusion vs Fission Mo99 production:
  - Understood that Mo99 production from U238 becomes more significant for fusion. Depleted Uranium is a lower risk material
- Mo99 production in a breeder blanket:
  - Mo99 production per neutron is largest in the rear of the blanket, where the neutrons are *slower*. But the absolute production rate is higher at the front where the flux is higher.
- Identification of all isotope production routes
  - Scans all nuclides/elements to model isotope production.
    - Help identify optimal production routes as a function of facility
  - Can be altered to include more spectra (and projectiles)
- Used the Modular Pincell as a concept isotope production device
  - Further refinement is required
  - Concept model of Mo99, Cu64, Lu177, Pb212 and Ac225 production. Which could be used for trials etc.





UK Atomic  
Energy  
Authority

# Radionuclides for Health UK

Jennifer Young, Barts Cancer Institute - Queen Mary  
University of London





**Barts**  
Cancer Institute

# Radionuclides for Health UK

Fusion for Medical Radionuclides UKAEA  
September 2025

Dr Jennifer Young

[jennifer.young@qmul.ac.uk](mailto:jennifer.young@qmul.ac.uk)



**CANCER  
RESEARCH  
UK**

City of  
London  
Centre

UCL, King's,  
Barts &  
the Crick



# Radionuclides For Health UK



**Dr Jane Sosabowski**  
Reader in Molecular Imaging  
Barts Cancer Institute



**Dr Jennifer Young**  
Postdoctoral Research Associate  
Barts Cancer Institute



**Prof Phil Blower**  
Professor of Imaging Chemistry, Head of  
Department of Imaging Chemistry and  
Biology



Engineering and  
Physical Sciences  
Research Council



Research  
England



RADNET  
CITY OF LONDON



UNIVERSITY OF  
COPENHAGEN



Department for  
Energy Security  
& Net Zero



UNIVERSITY OF  
CAMBRIDGE



The British  
Institute of  
Radiology



Barts  
Cancer Institute



The Institute of  
Cancer Research

Imperial College  
London



Neuroendocrine  
Cancer UK



ImaginAb



CANCER  
RESEARCH  
UK

CENTRE  
FOR DRUG  
DEVELOPMENT



UNIVERSITY OF  
BIRMINGHAM



CANCER  
RESEARCH  
UK

BEATSON  
INSTITUTE



The ROYAL MARSDEN  
NHS Foundation Trust

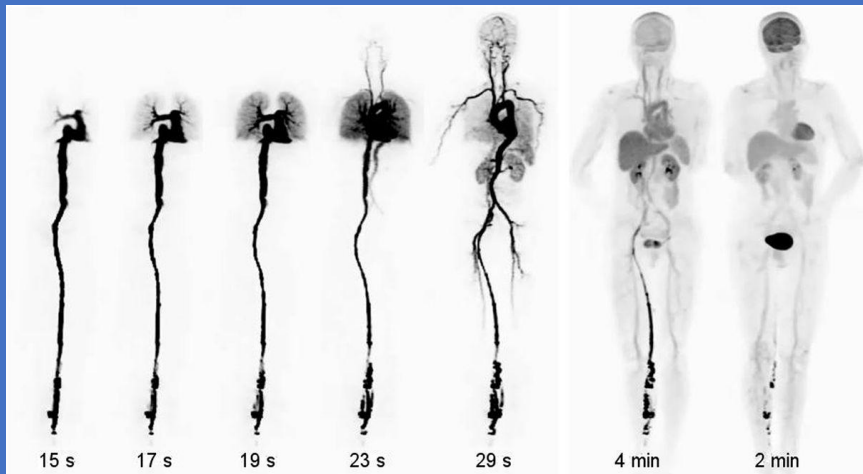
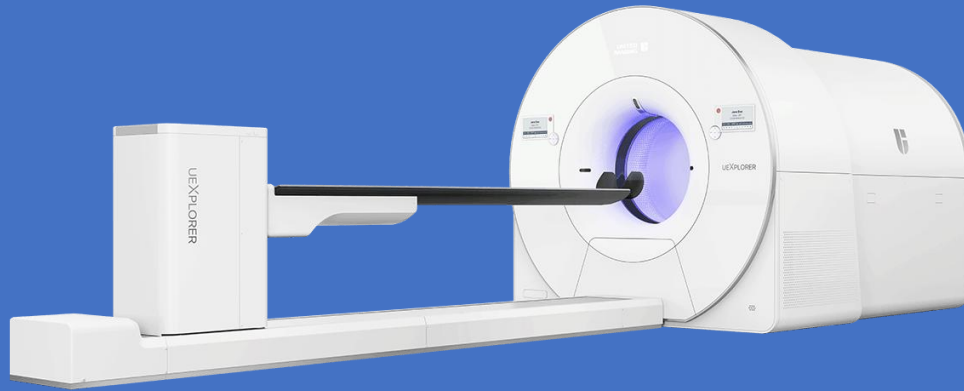




# Nuclear Medicine Resurgence

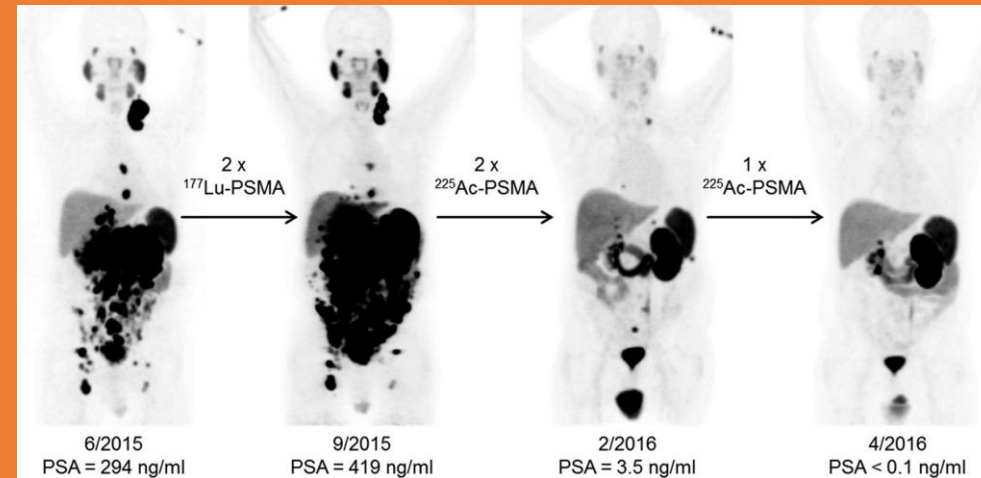
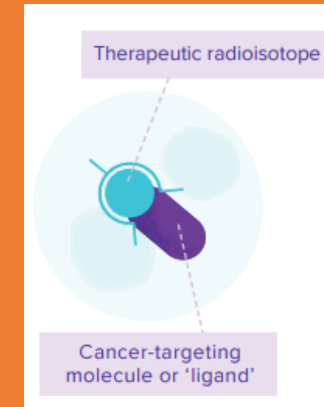
Driven by two complementary areas

## Total Body Positron Emission Tomography (PET)



Journal of Nuclear Medicine March 2019, 60 (3) 299-303

## Molecular Radiotherapy



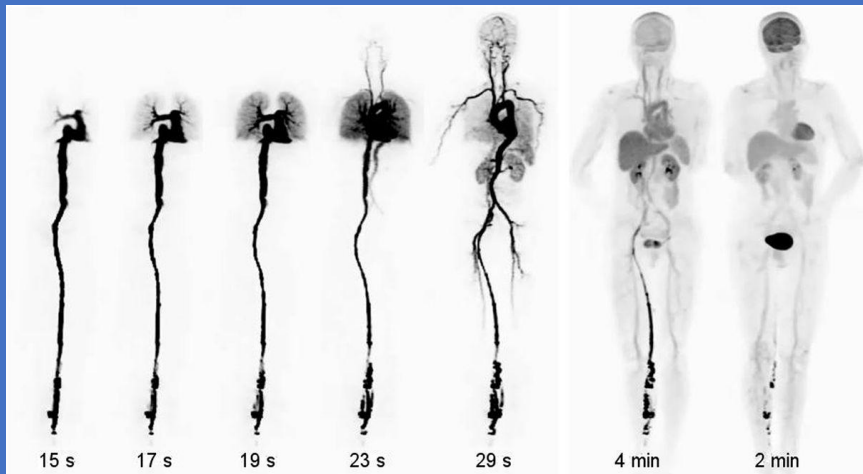
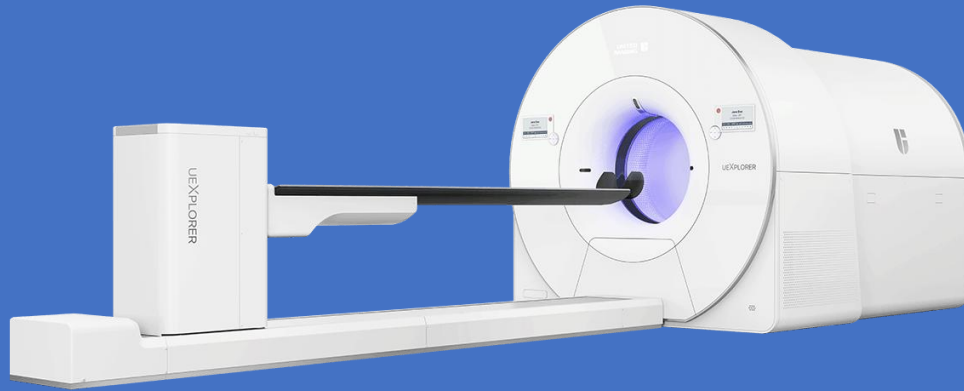
Kratochwil et al, J Nucl Med 2016; 57:1941–1944



# Nuclear Medicine Resurgence

Driven by two complementary areas

## Total Body Positron Emission Tomography (PET)

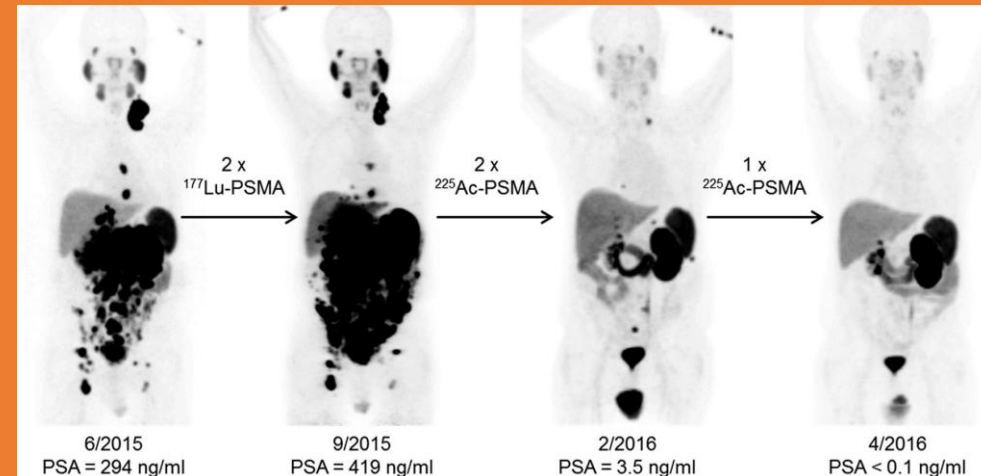


Journal of Nuclear Medicine March 2019, 60 (3) 299-303



With an investment of £32 million from the UK Government through UK Research and Innovation (UKRI), NPIP's ground breaking total-body PET technology platform will help drive the nation's reputation as a global life science superpower.

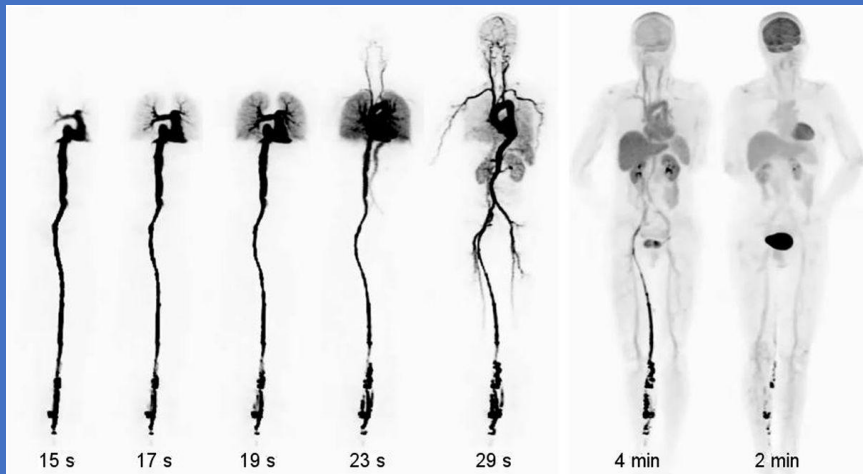
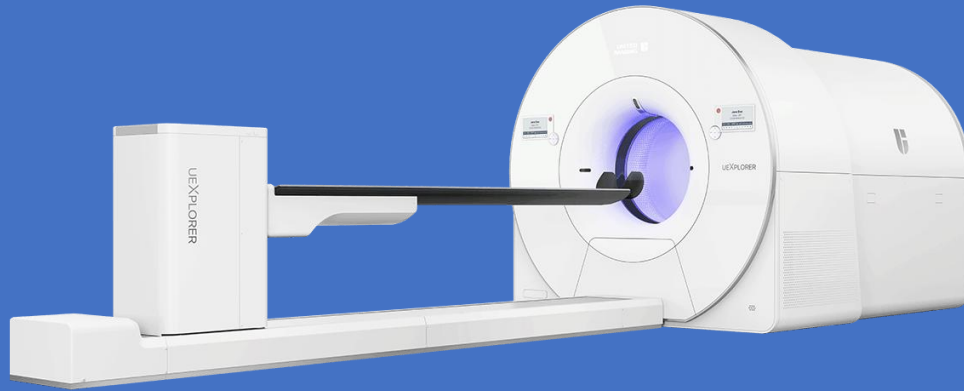




# Nuclear Medicine Resurgence

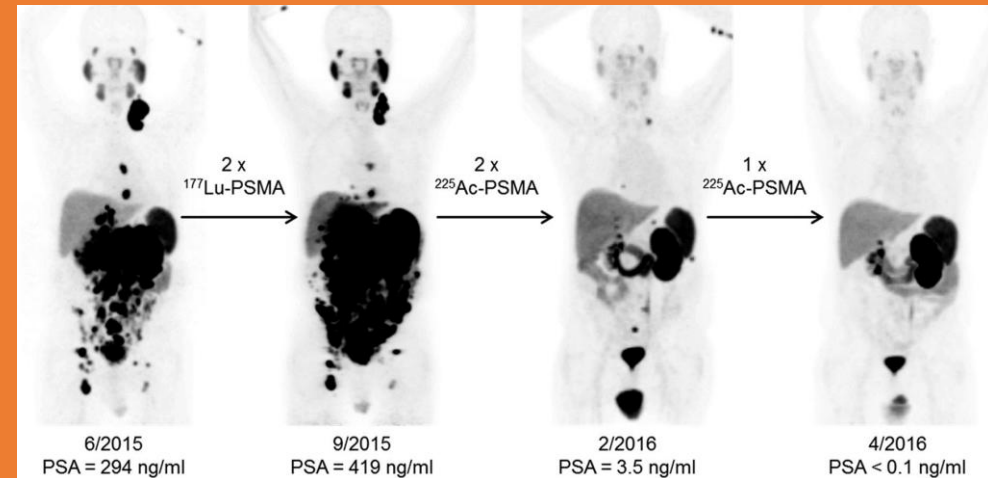
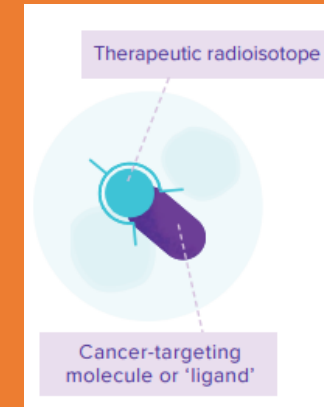
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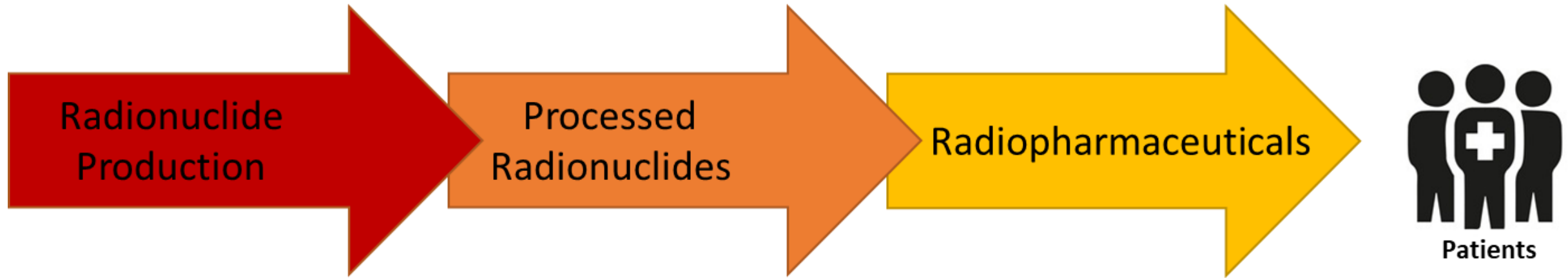


Kratochwil et al, J Nucl Med 2016; 57:1941–1944



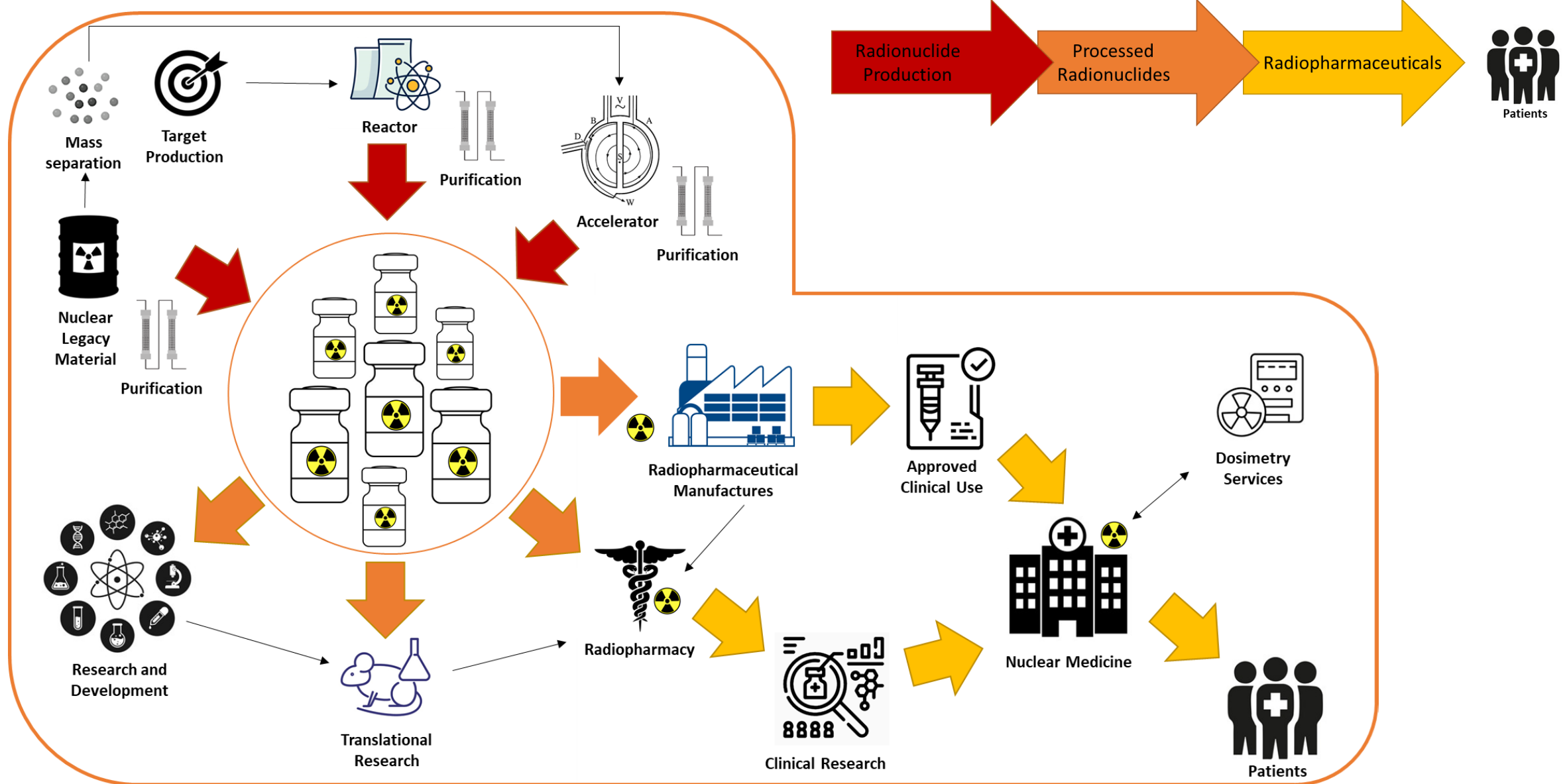
# Vision for UK

From Radionuclide Production to Molecular Radiotherapy for Patients



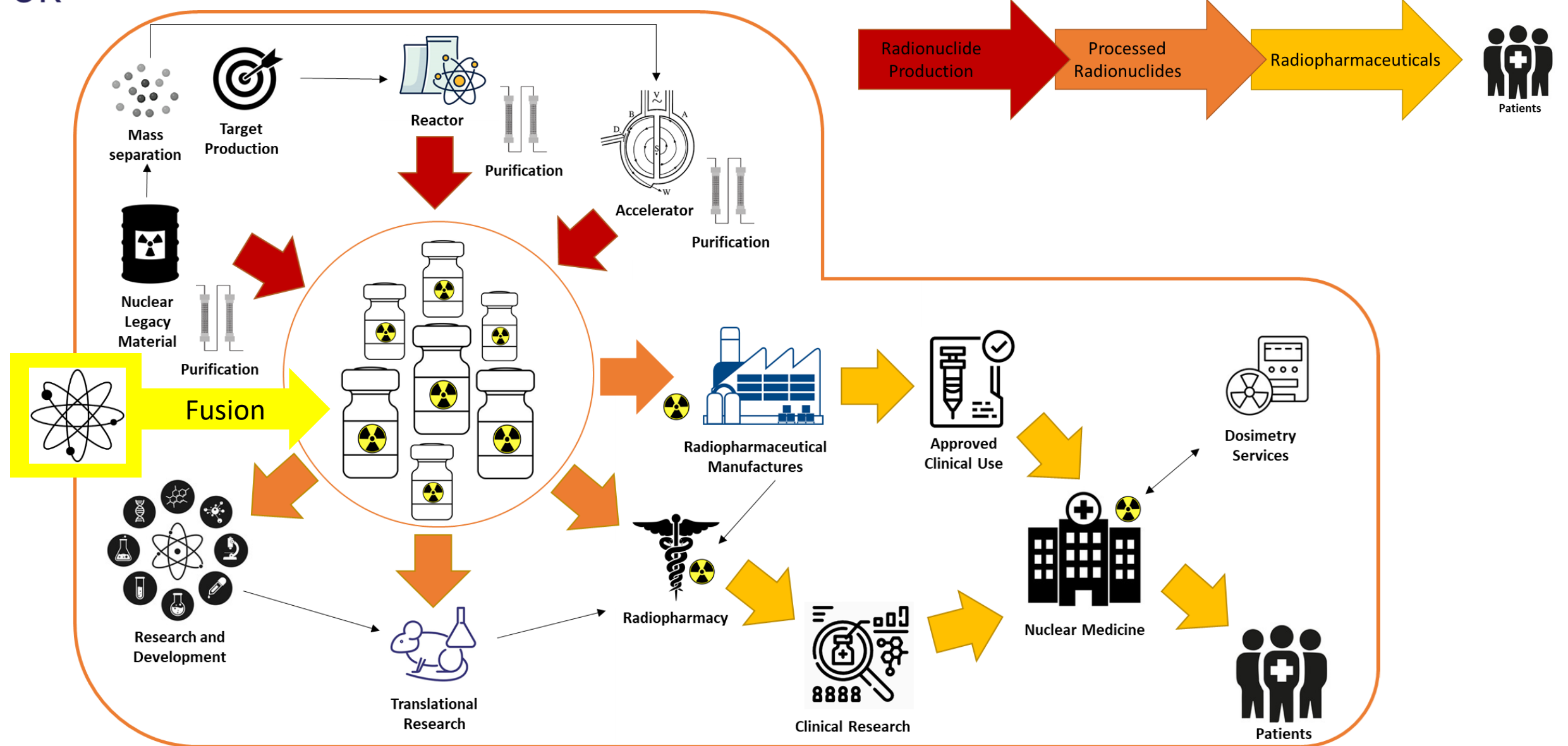
## Vision for UK

From Radionuclide Production to Molecular Radiotherapy for Patients



# Vision for UK

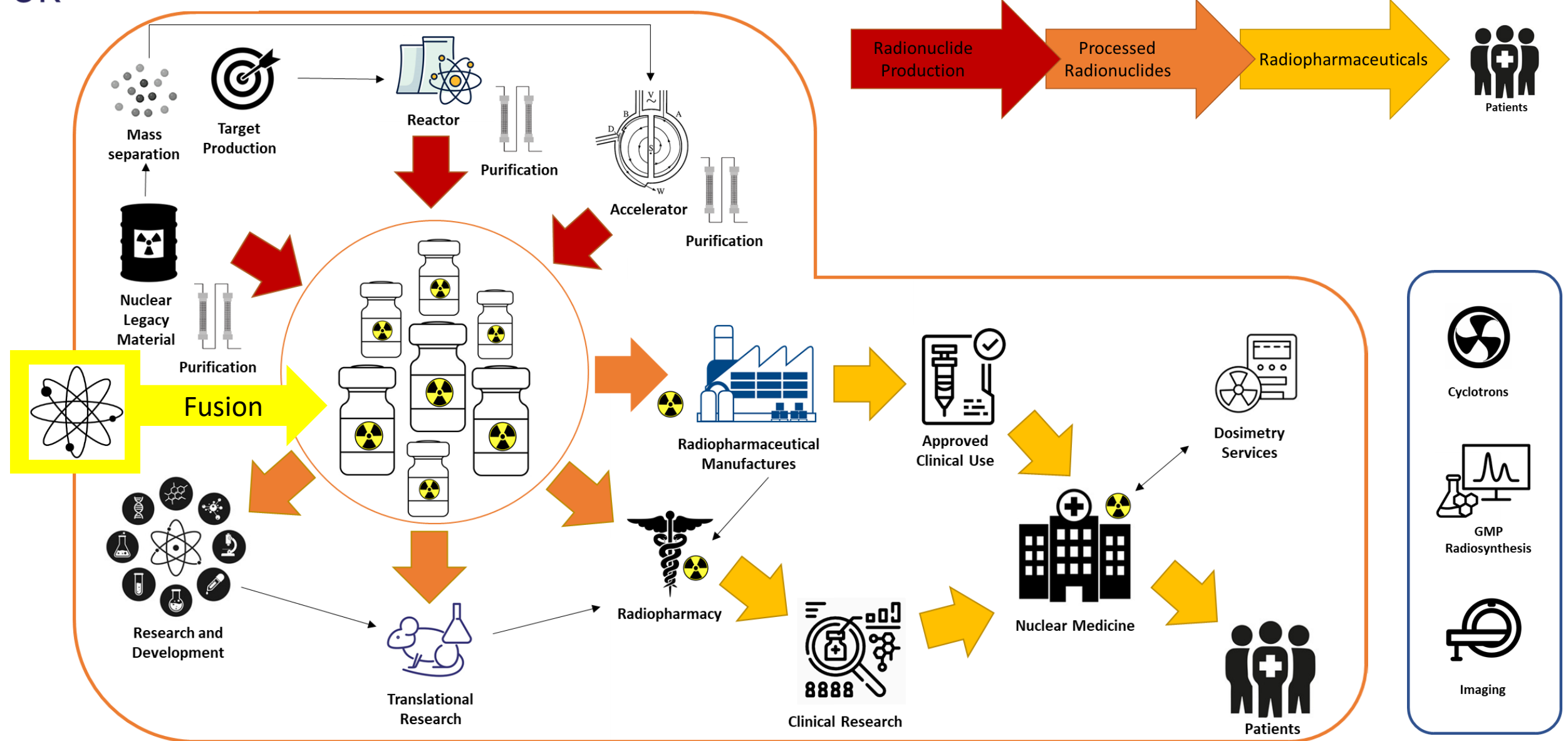
From Radionuclide Production to Molecular Radiotherapy for Patients





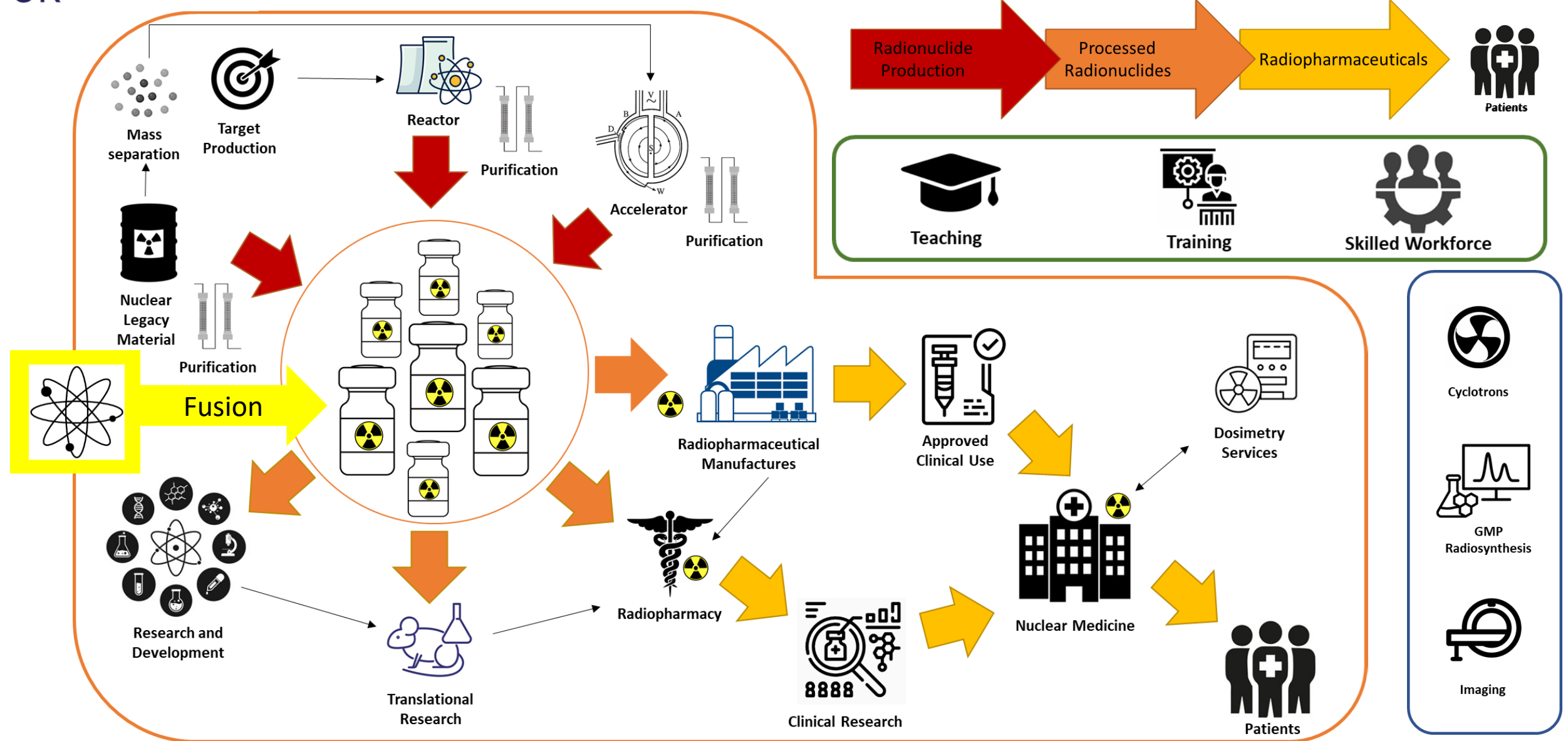
# Vision for UK

From Radionuclide Production to Molecular Radiotherapy for Patients

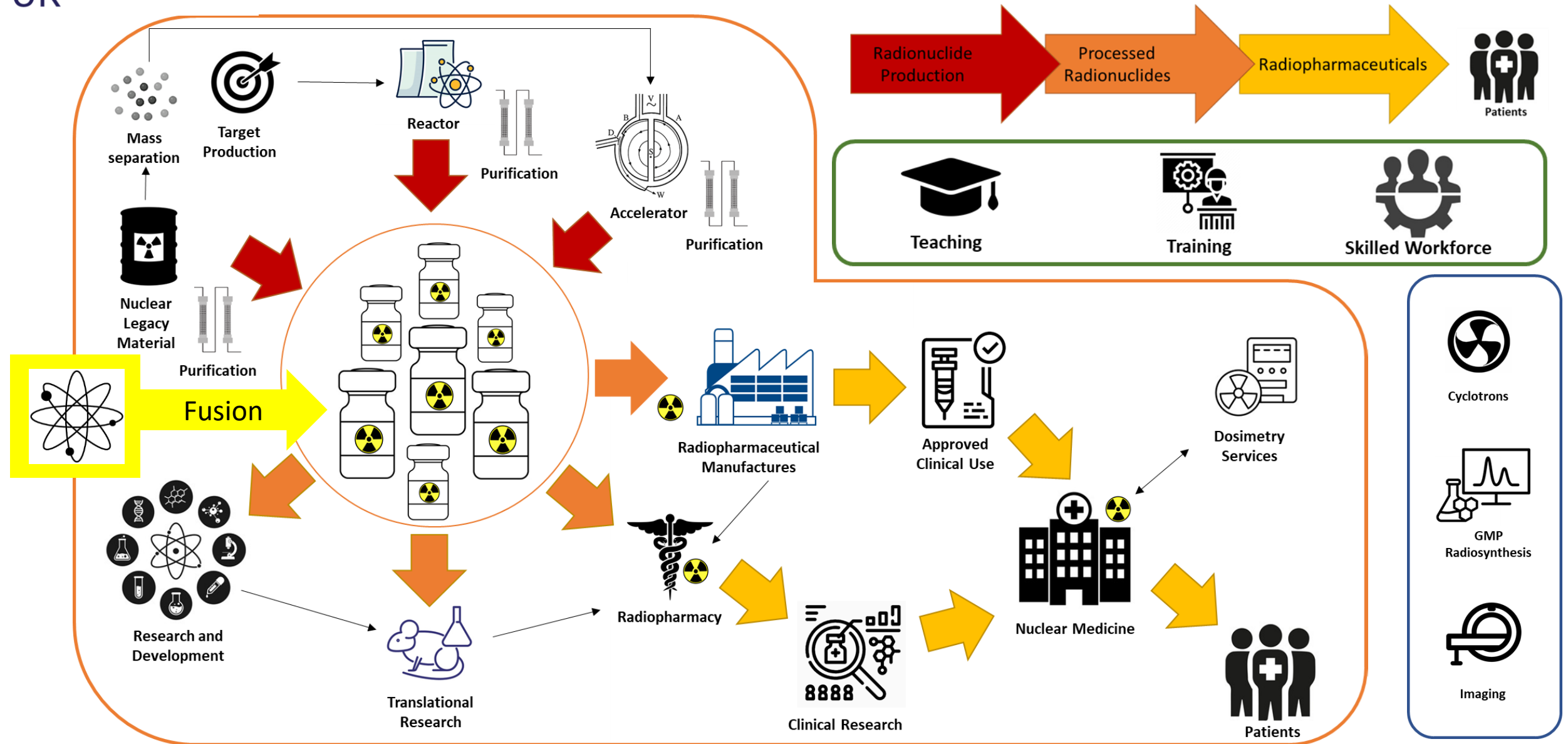


## Vision for UK

From Radionuclide Production to Molecular Radiotherapy for Patients

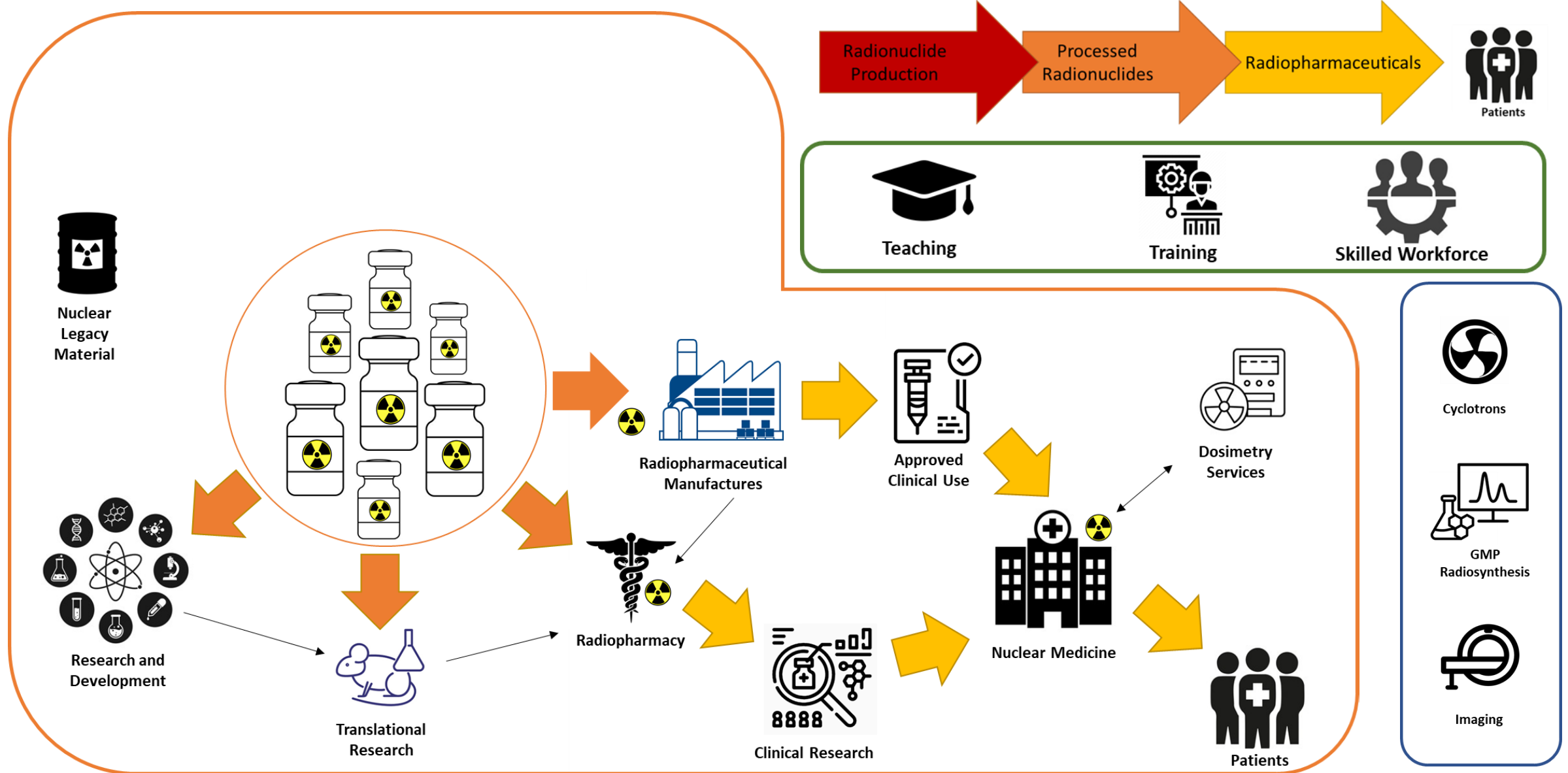


# What do we have now?

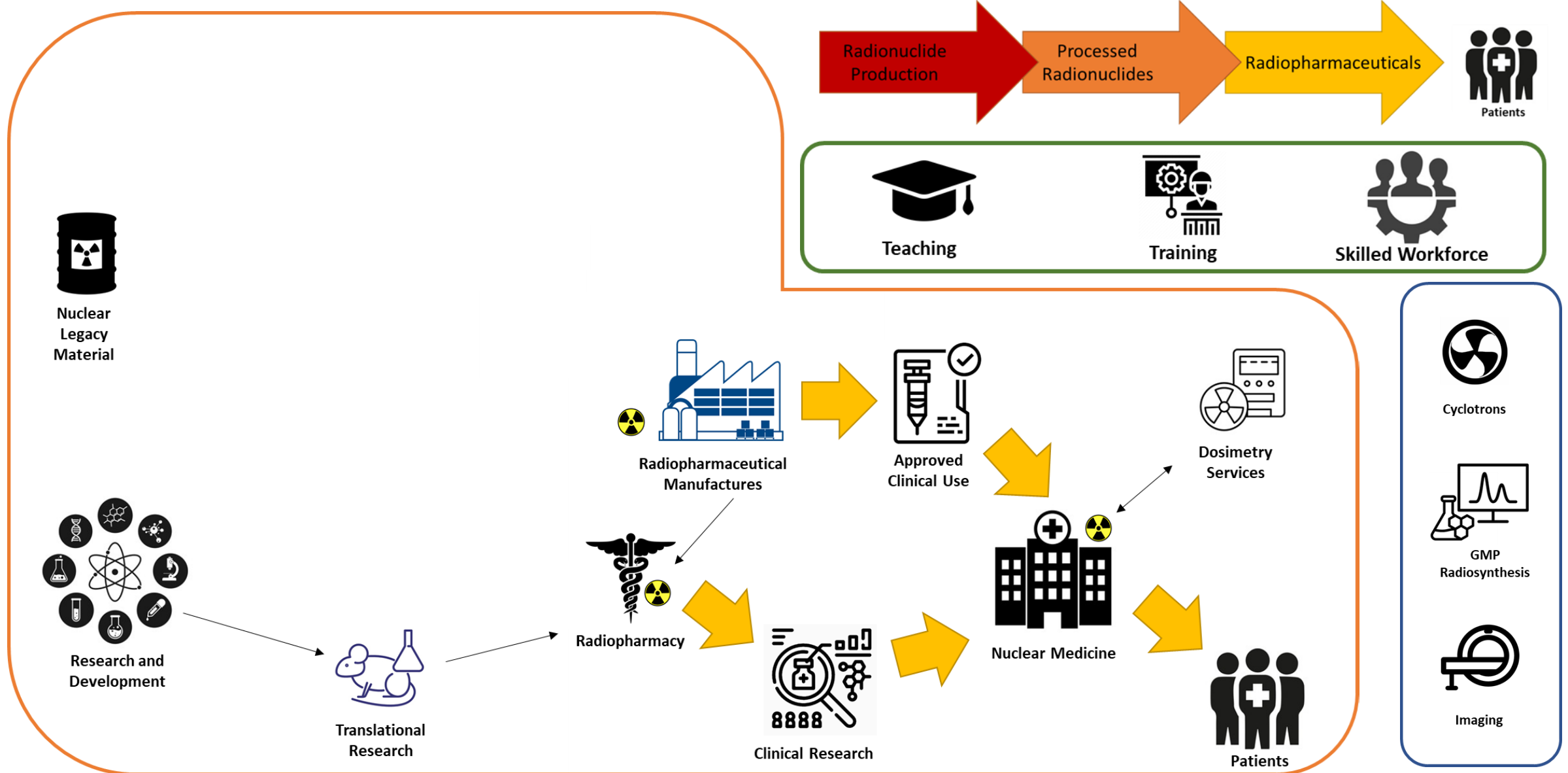




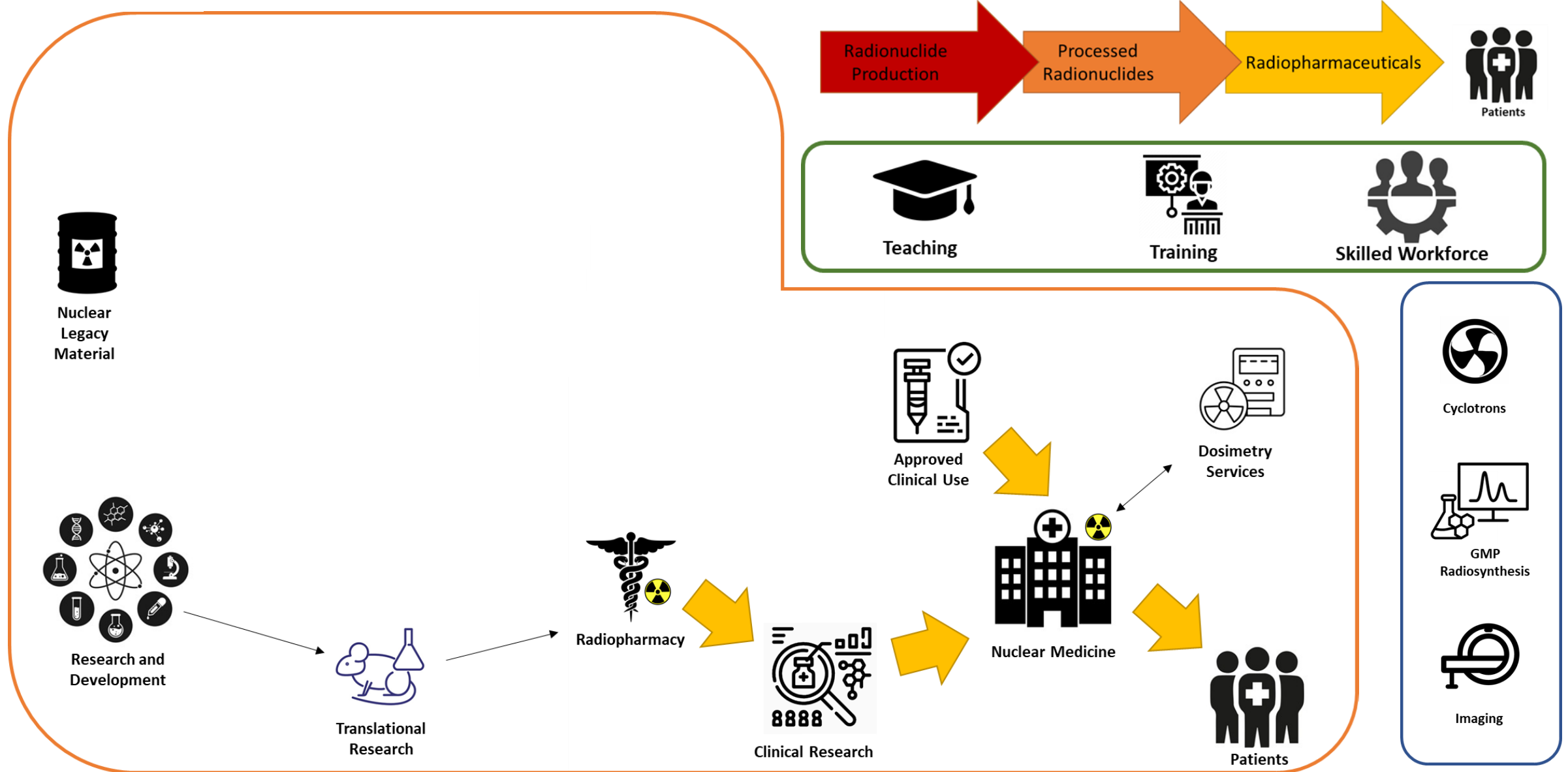
## What do we have now?



# What do we have now?

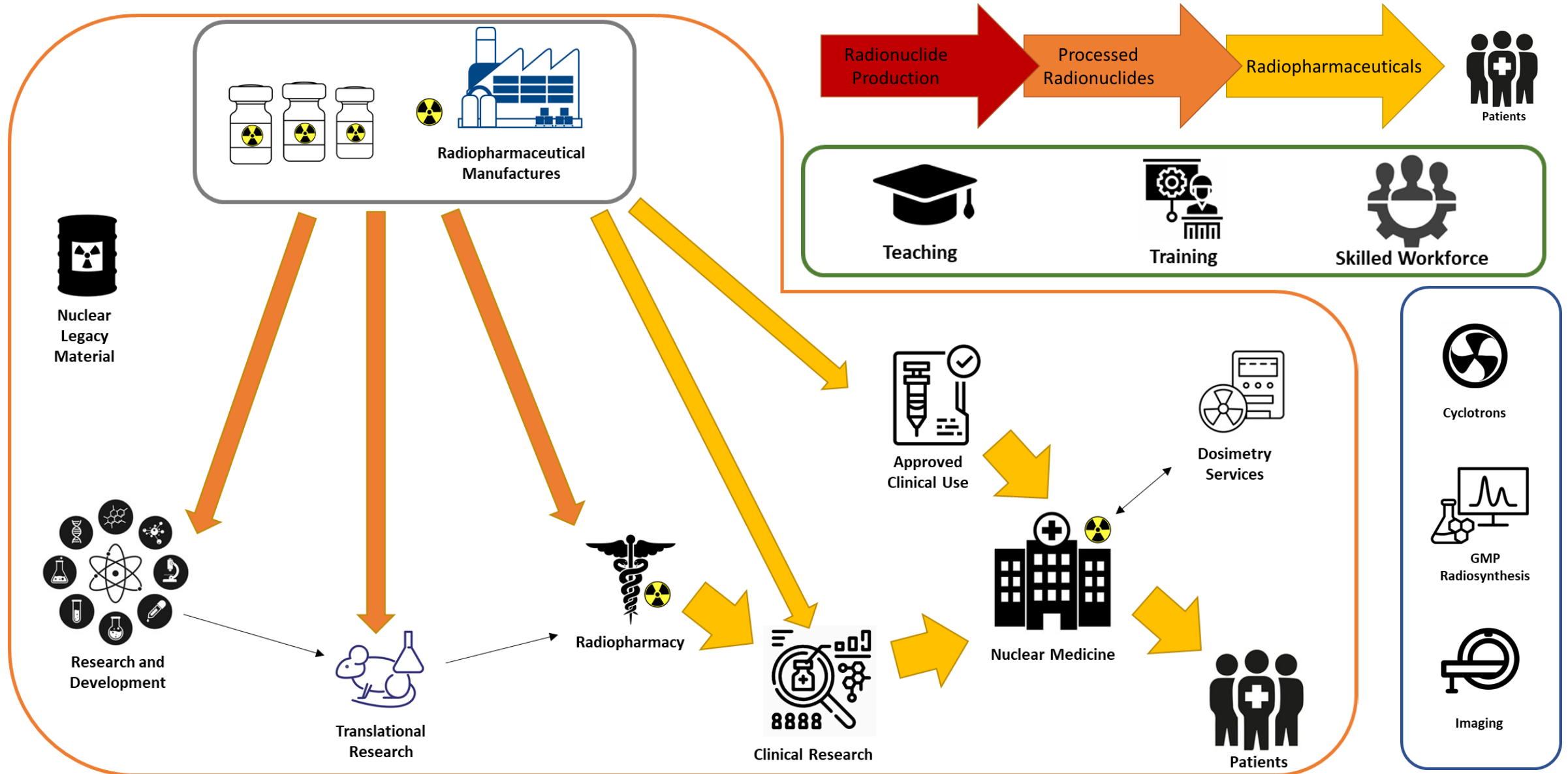


# What do we have now?

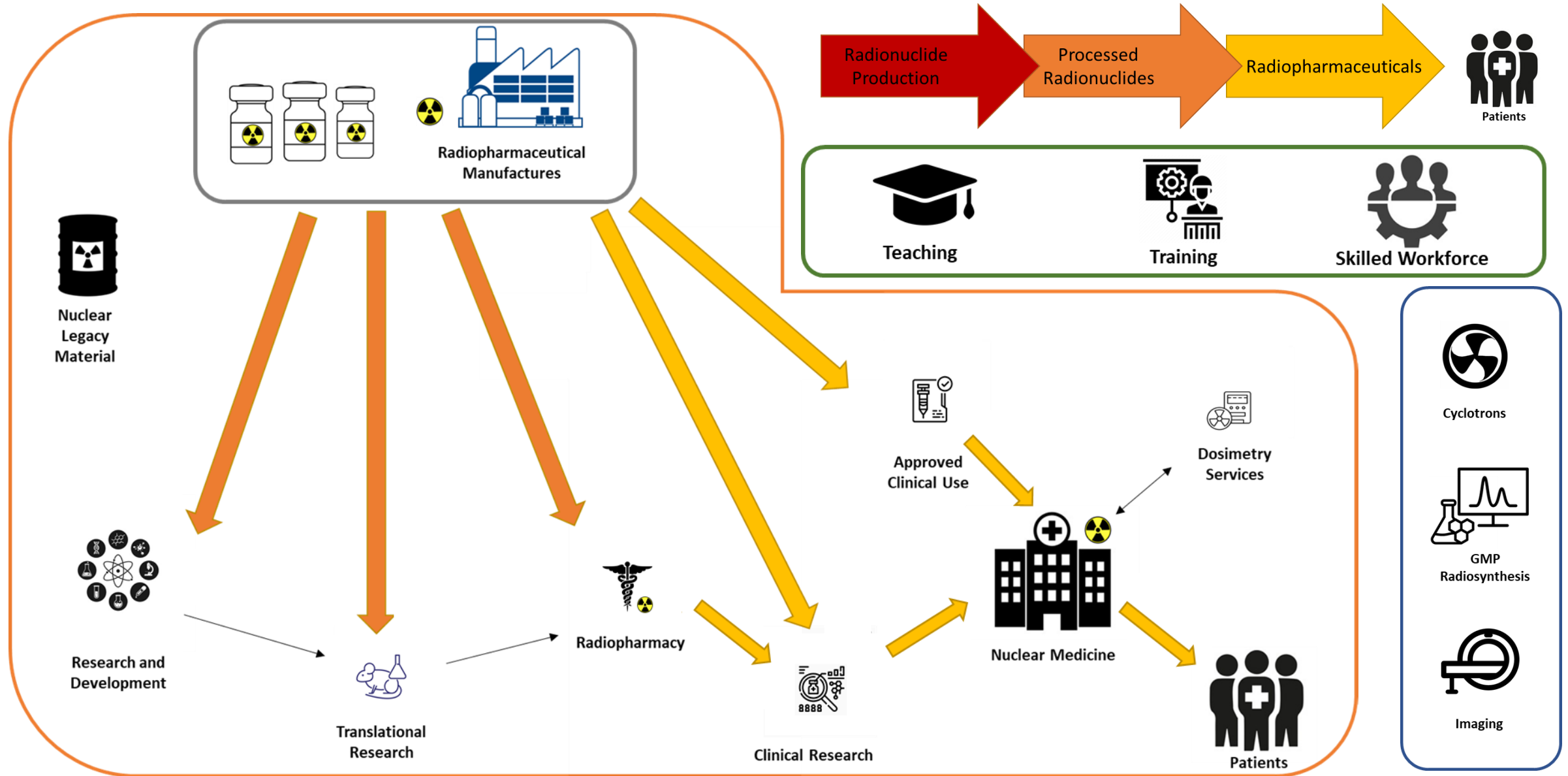




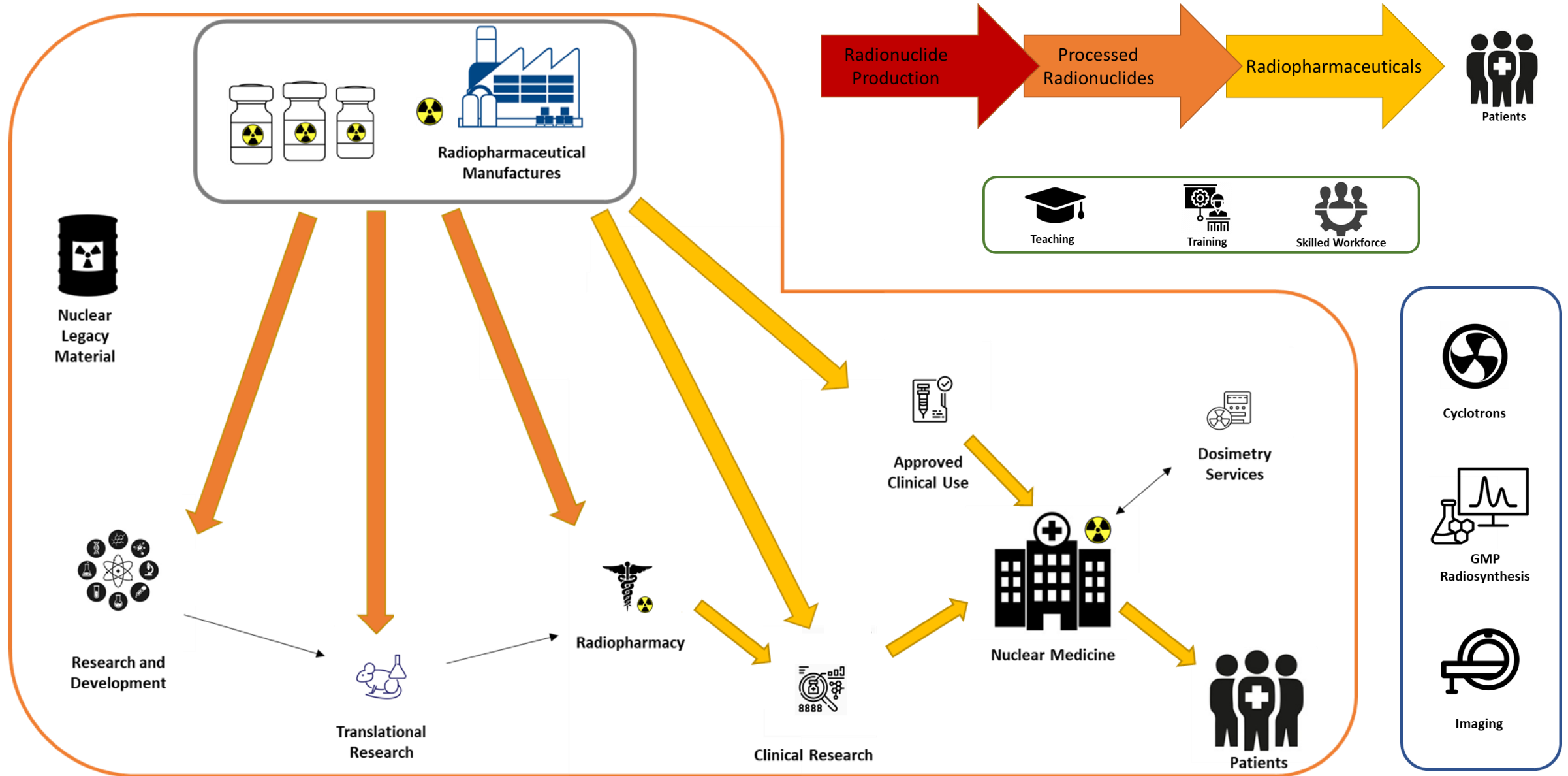
# What do we have now?



# What do we have now?



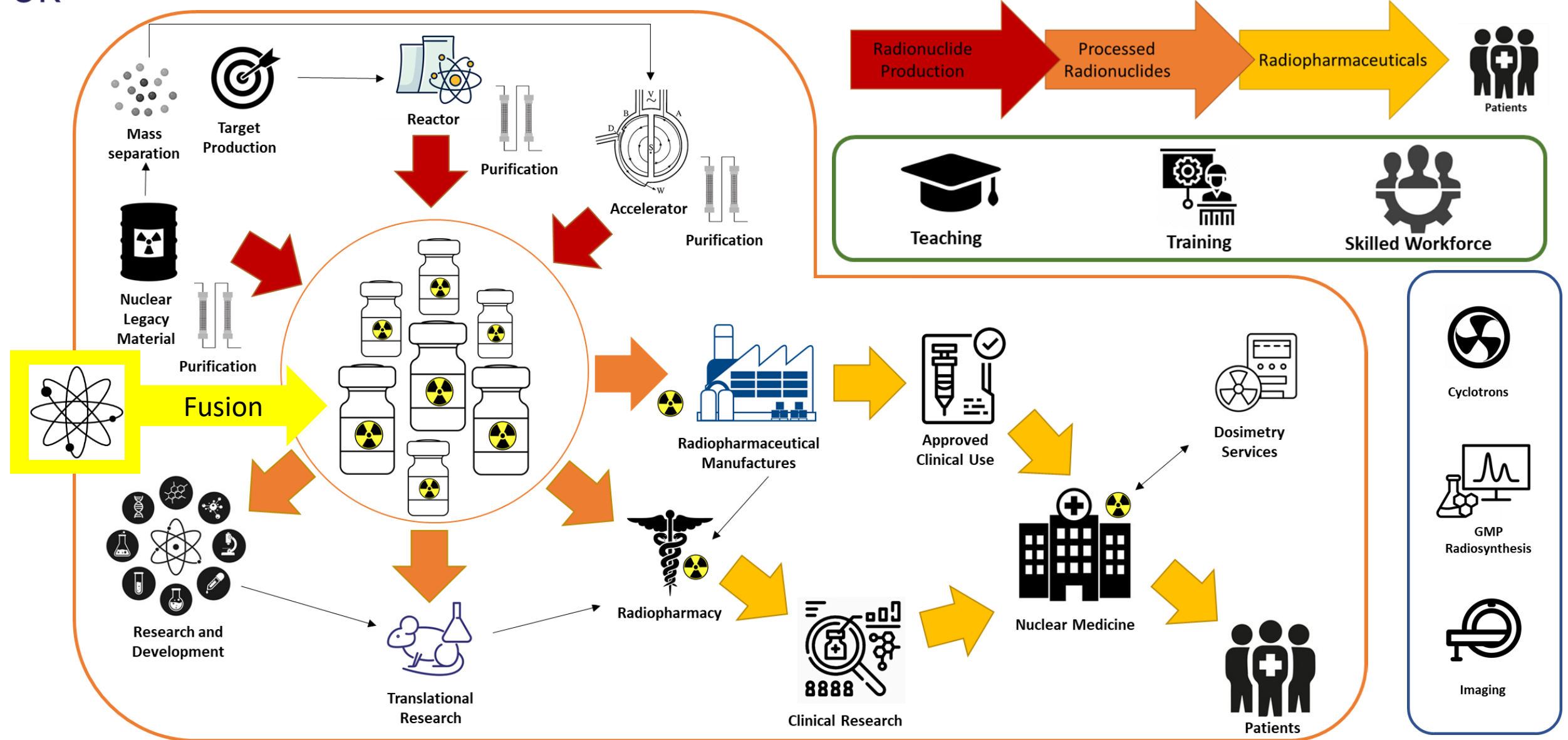
## What do we have now?





## Vision for UK

From Radionuclide Production to Molecular Radiotherapy for Patients



# Radionuclides For Health UK

## UK Initiatives to Improve Radionuclide Supply



Radionuclides  
For Health  
UK

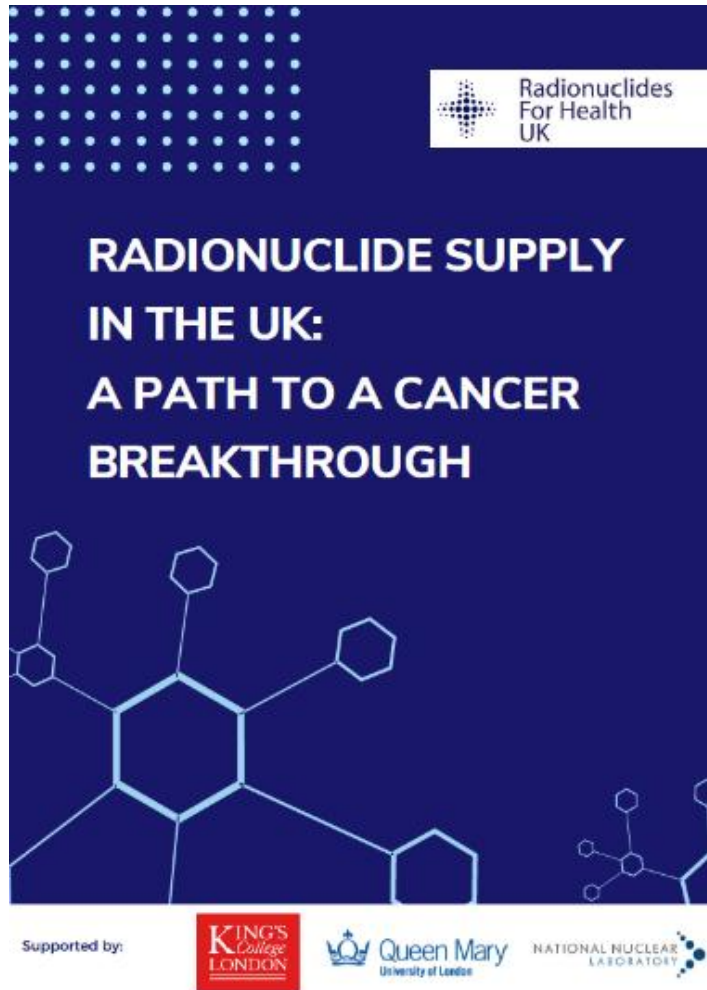


Medical  
Radionuclide  
Innovation  
Programme



Llywodraeth Cymru  
Welsh Government

Advanced Radioisotope Technology  
for Health Utility Reactor (ARTHUR)



1. Nuclear infrastructure
2. Processing capabilities
3. Regulatory framework
4. Centralised radiopharmacy
5. Skills gap

<https://www.bartscancer.london/radionuclides-for-health-uk/>

## Research Projects

At-211

I-124

Lead-212





Department for  
Energy Security  
& Net Zero



Dept for BEIS

@beisgovuk

Official

Yesterday, Chief Scientific Advisor @psmonks visited @HuMIC\_ to launch a £6m Medical Radionuclide Innovation Programme – a new scheme developing technologies which will produce nuclear materials for diagnostics and cancer therapies #InnovationNation



Medical  
Radionuclide  
Innovation  
Programme

### Awarded Innovation Projects

Queen Mary University of London:  
Development of UK **Astatine-211** Production  
Capability

Cyclife Aquila Nuclear: Hot Cells to House a  
**Lead-212** Generator for Integration Into a  
Hospital Environment

King's College London: Development of UK  
**Iodine-124** Production Capability

National Physical Laboratory: Research of the  
End-to-End Accelerator Production of  
**Actinium-225** from Radium-226

Urenco: Development of UK Supply Chain for  
**Copper** Medical Isotopes

Dalton Cumbrian Facility: Optimised  
Production of Theragnostic Isotopes of  
**Copper and Scandium**

National Nuclear Laboratory: Investigating  
the Recovery of Strontium-90 From Legacy  
Nuclear Material for the Sustained Production  
of **Yttrium-90** for Targeted Therapies

National Nuclear Laboratory: Sustainable  
Production of Radionuclides for **Targeted  
Alpha Therapy** from UK Stocks of Recycled  
Uranium

National Nuclear Laboratory: Accelerated  
Supply of Radionuclides for Cancer Treatment  
by Developing Protactinium and **Actinium**  
Separation Techniques

National Physical Laboratory: Medical  
Radionuclide Production by Laser Wakefield  
Accelerator



Llywodraeth Cymru  
Welsh Government

PRESS RELEASE

## Welsh Government unveils major plans for national nuclear medicine laboratory in north Wales

Major new plans to make Wales a global centre of excellence and the leading location for medical radioisotope production in the UK, which would help address a fast-approaching supply crisis for nuclear medicine around the world have been unveiled today by the Welsh Government.

First published: 10 January 2023

Last updated: 10 January 2023

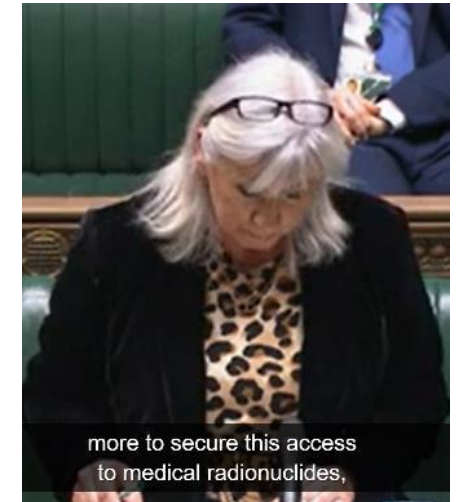
- Welsh Government's Project ARTHUR would see the creation of a public sector national laboratory for the supply of medical radioisotopes, needed for the diagnosis and treatment of diseases such as cancer.
- Facility would be a global centre of excellence in nuclear medicine, making Wales the leading location for medical radioisotope production in the UK.
- Development will lead to the creation of highly skilled jobs over several decades.
- Economy Minister calls on the UK Government to help fund the project to avoid a "future health and economic crisis."



In the UK, across Europe, and further afield, the equipment in facilities

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Optioneering Technical Report	Completed in 2020
Office for Nuclear Regulations Study	Completed in 2021
Strategic Outline Business Case	Completed in 2021
Supply and Demand Study	Completed in 2022
Feasibility Study	Completed in 2024
Outline Business Case	Ongoing



Adjournment debate in UK Parliament on the 22<sup>nd</sup> Feb 2023



SCIENCE

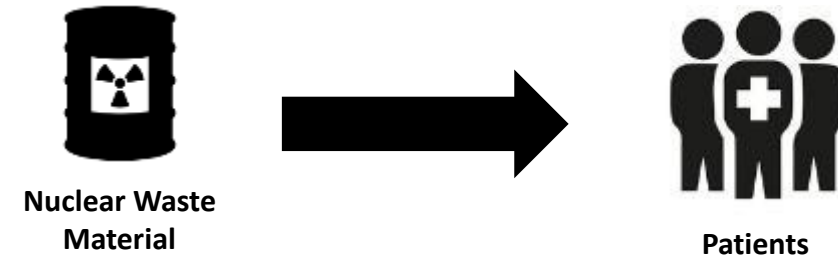
## Nuclear waste could be used to target cancer cells

Kat Lay, Health Editor

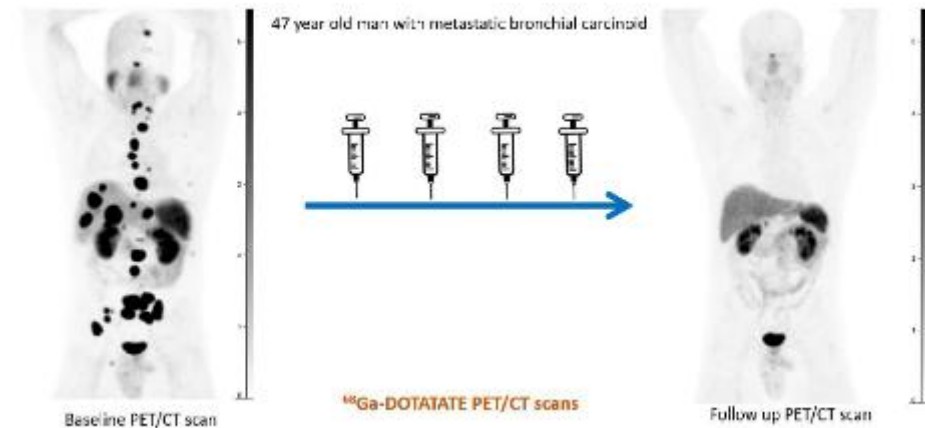
Monday May 30 2022, 1:20pm,  
The Times



A production process has been developed that could mean waste from nuclear power stations such as Sizewell in Suffolk can be put to good use  
ALAMY



Phase 1 clinical trial of Alpha particle PRRT with  $^{212}\text{Pb}$ -DOTAMTATE

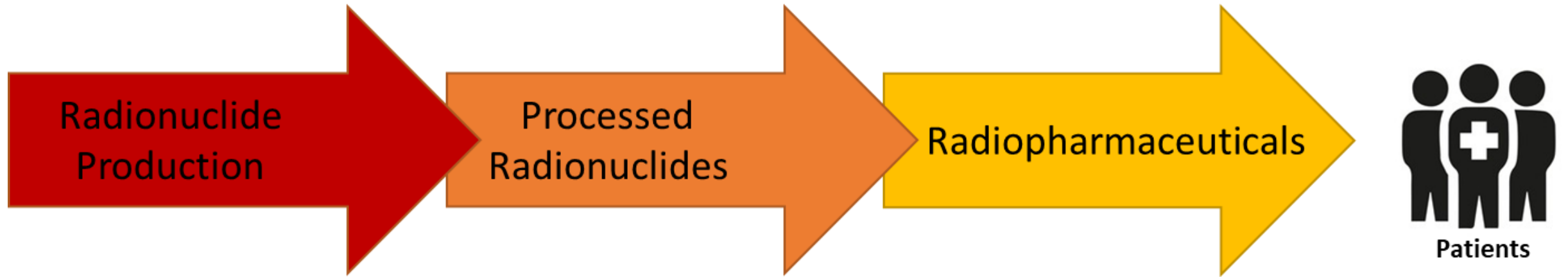


<https://jnm.snmjournals.org/content/early/2022/01/06/jnumed.121.263230>



# Vision for UK

From Radionuclide Production to Molecular Radiotherapy for Patients



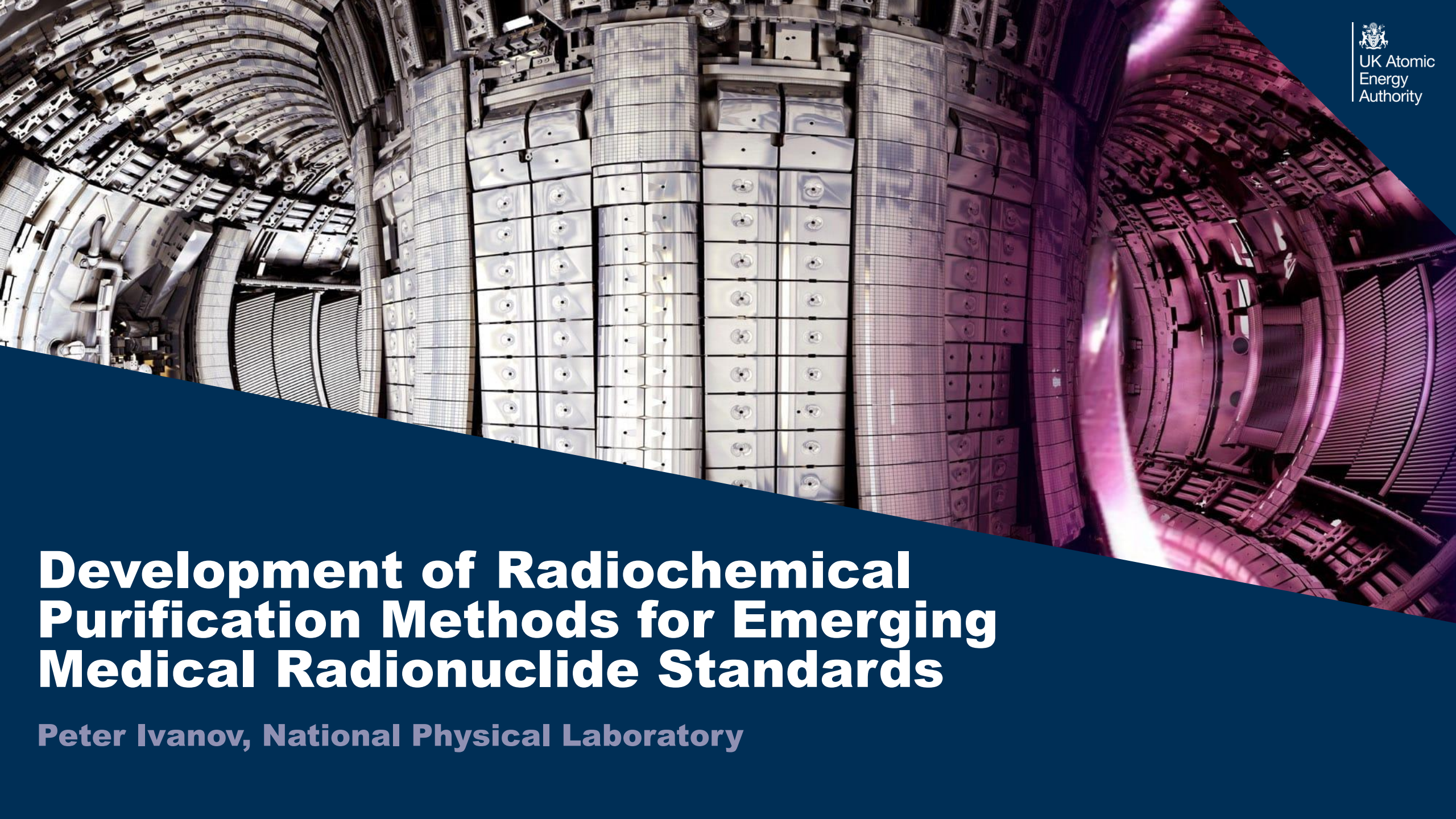
# Thank You



# Coffee Break

See you at 11:45





# **Development of Radiochemical Purification Methods for Emerging Medical Radionuclide Standards**

**Peter Ivanov, National Physical Laboratory**

# Development of radiochemical purification methods for emerging medical radionuclide standards

Peter Ivanov

National Physical Laboratory





**A National Laboratory and the UK's National Measurement Institute (DSIT owned Public Corporation)**



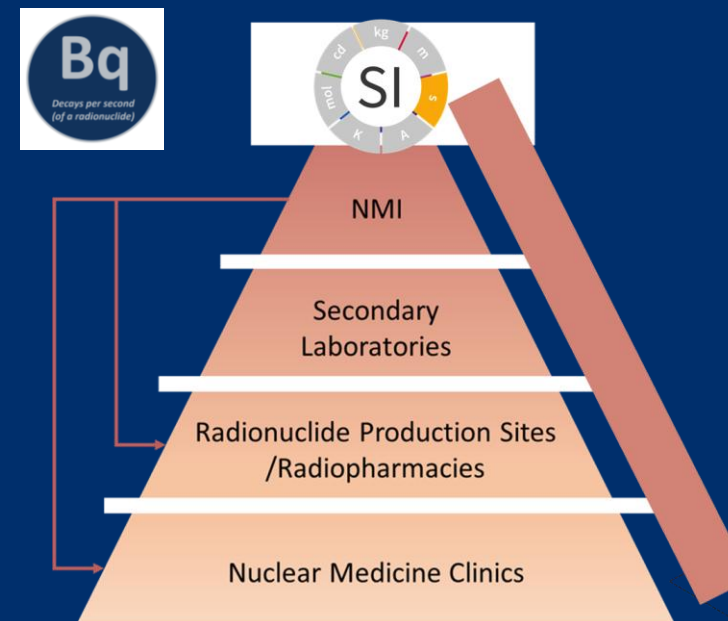
**A key part of the UK 's nuclear medicine infrastructure providing metrology support across value chain for more than 100 years.**



**Expertise includes traceable radioactivity and radiochemistry measurements, imaging and therapeutic dosimetry.**

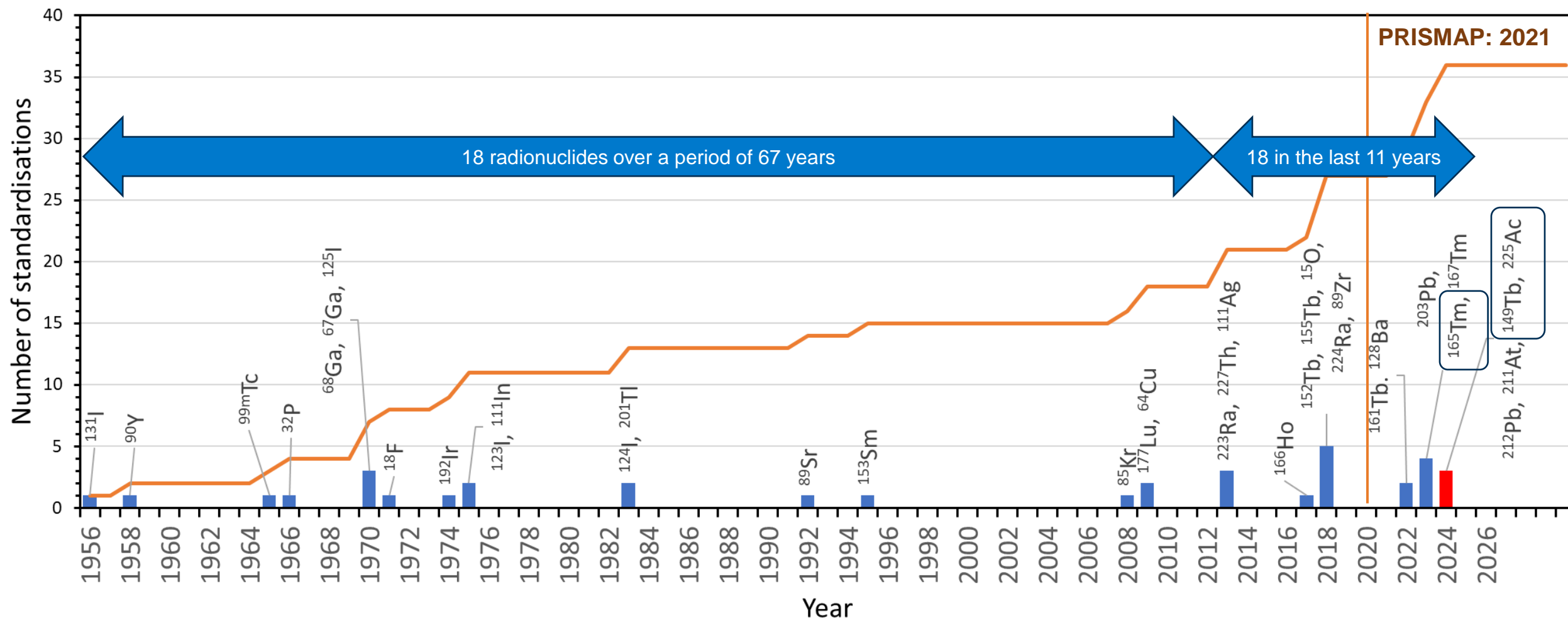


**Strategic partnerships with industry, government, academia, charities and regulators**





# Medical Radionuclide Standardisation at NPL



PRISMAP (CERN-MEDICIS) collaboration has greatly increased the number of medical radionuclide primary standards prepared by NPL

# Comprehensive metrology for nuclear medicine

## Stakeholders

Pharmaceutical  
Companies

National  
Regulators

Standards  
Bodies

Medical Imaging  
Manufacturers

Hospitals

## Clinical Radiopharmaceutical Pathway

Production and  
Supply

Radiopharmacy

Secondary  
Standards

Quantitative  
Imaging

Dosimetry (MRT)

Radiochemistry

Primary  
Standards

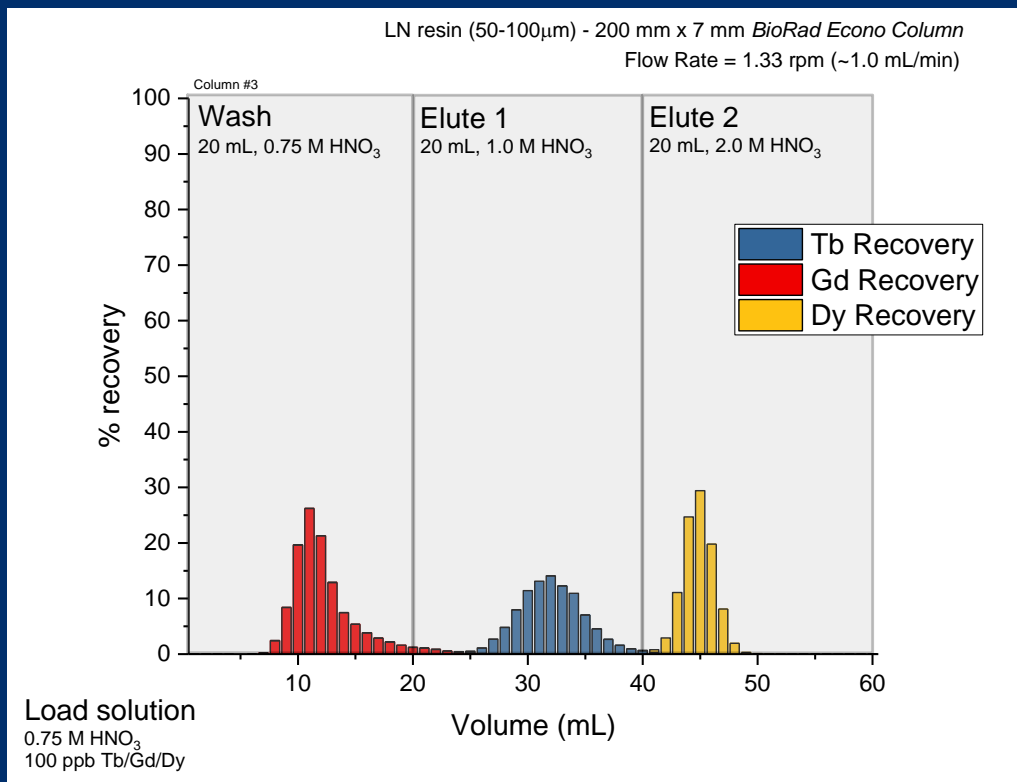
Nuc. Medicine  
Imaging

Nuclear Data

Dosimetry  
Calculations

## Metrology

# Radiochemical separations



- Provide high-purity sources for standardisation and nuclear data measurements
- Better binding efficiency of targeting molecule by removing long-lived or stable impurities
- Reduced side-effects from radioactive or stable impurities
- Increased specific activity
- Less imaging interferences (SPECT/PET)



# Case study: Theranostic Terbium Isotopes

**$^{149}\text{Tb}$**

Alpha therapy

4.12 h

**$^{152}\text{Tb}$**

PET diagnostics

17.5 h

**$^{155}\text{Tb}$**

SPECT diagnostics

5.32 d

**$^{161}\text{Tb}$**

Beta/Auger therapy  
SPECT diagnostics

6.89 d

- Proton-induced spallation reaction on a Tantalum target
- On-line mass separation at 155 m/q

Production Facility –  
CERN MEDICIS

p-reaction  
on Ta target



Radiochemistry – NPL  
Preparation of target  
radionuclide

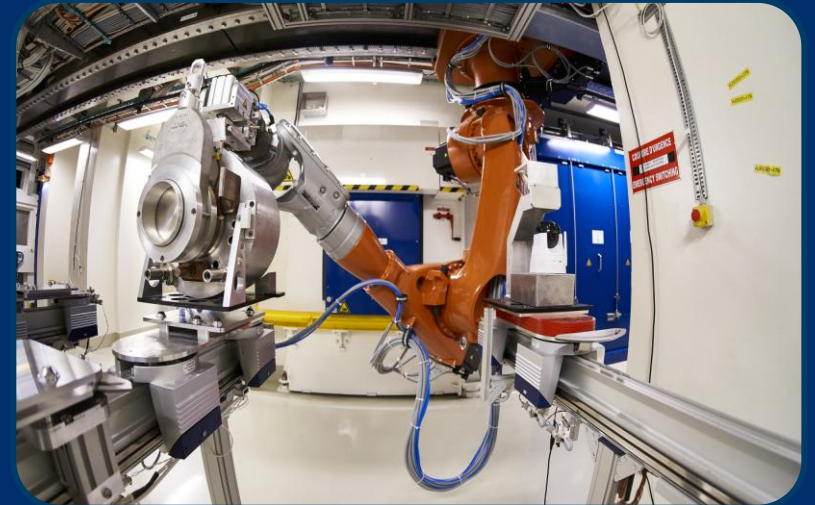


Contaminant assessment +  
nuclear decay data  
measurements– NPL

Ce

Tb

Gamma spectrometry



# $^{155}\text{Tb}$ Purification

- Selective oxidation of cerium
- Chromatographic separation on TEVA resin



## Selective oxidation of cerium:

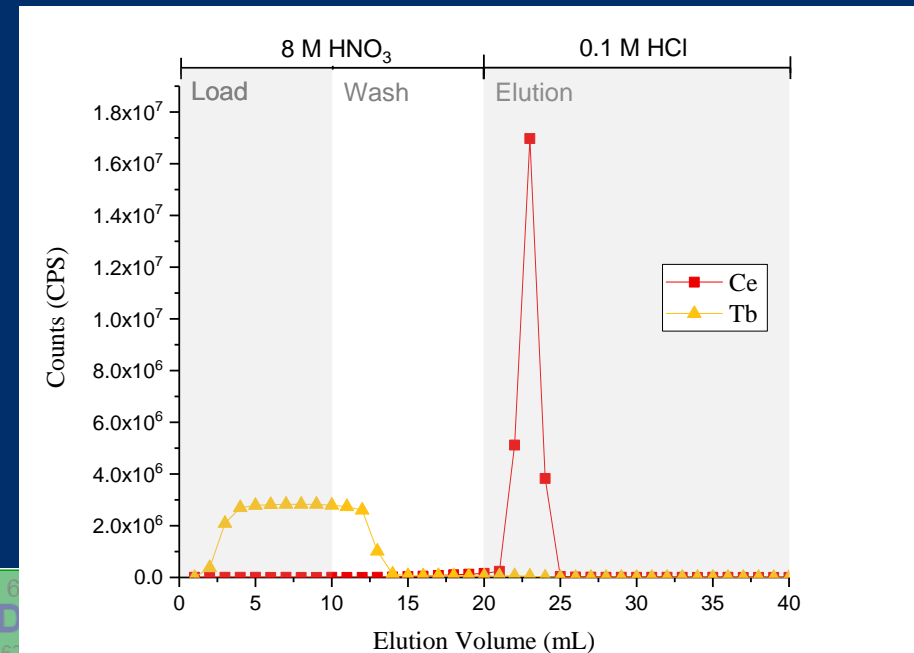
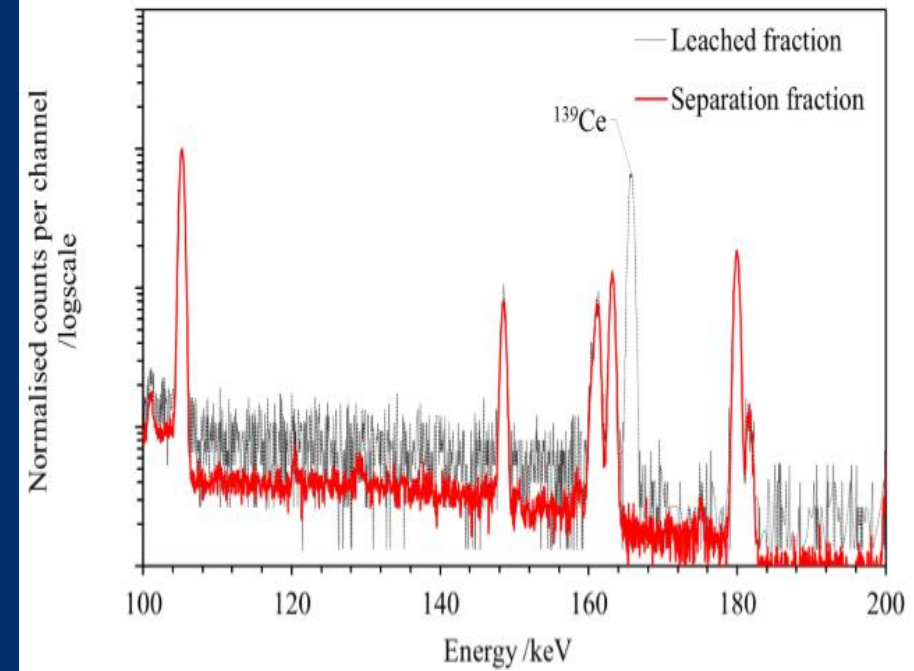
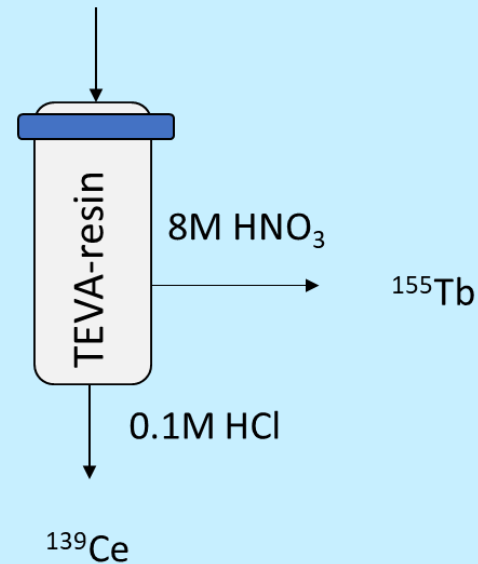


Supplied  $^{155}\text{Tb} = \sim 8.1 \text{ MBq}$

$^{139}\text{Ce}$  before  $\sim 31\%$

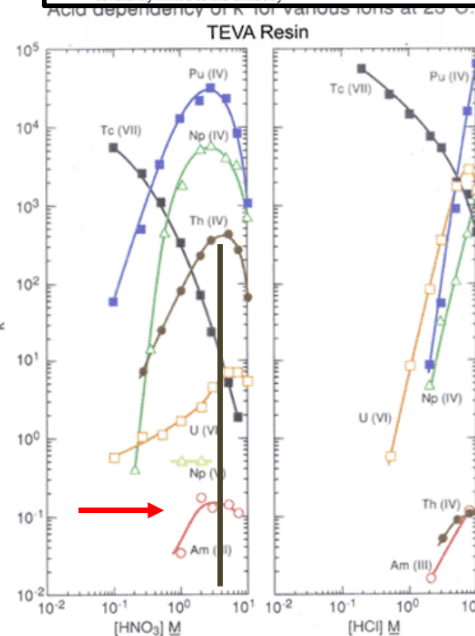
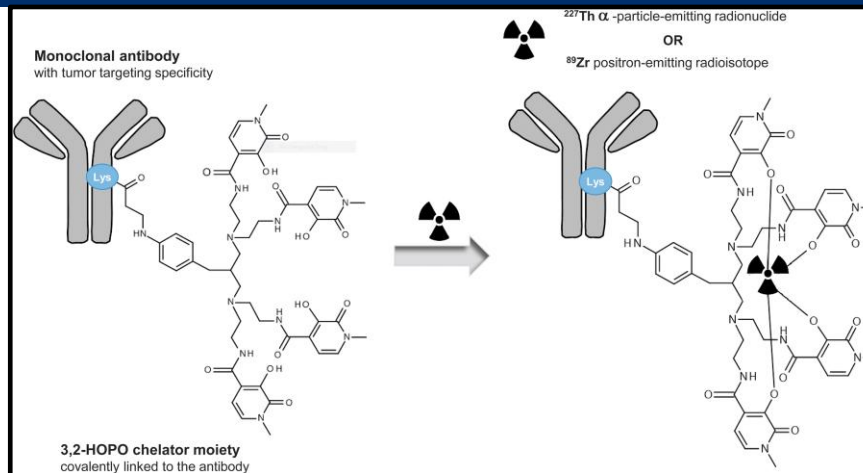
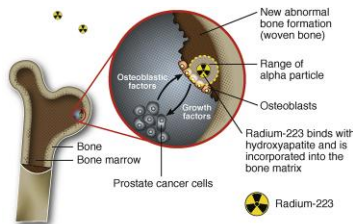
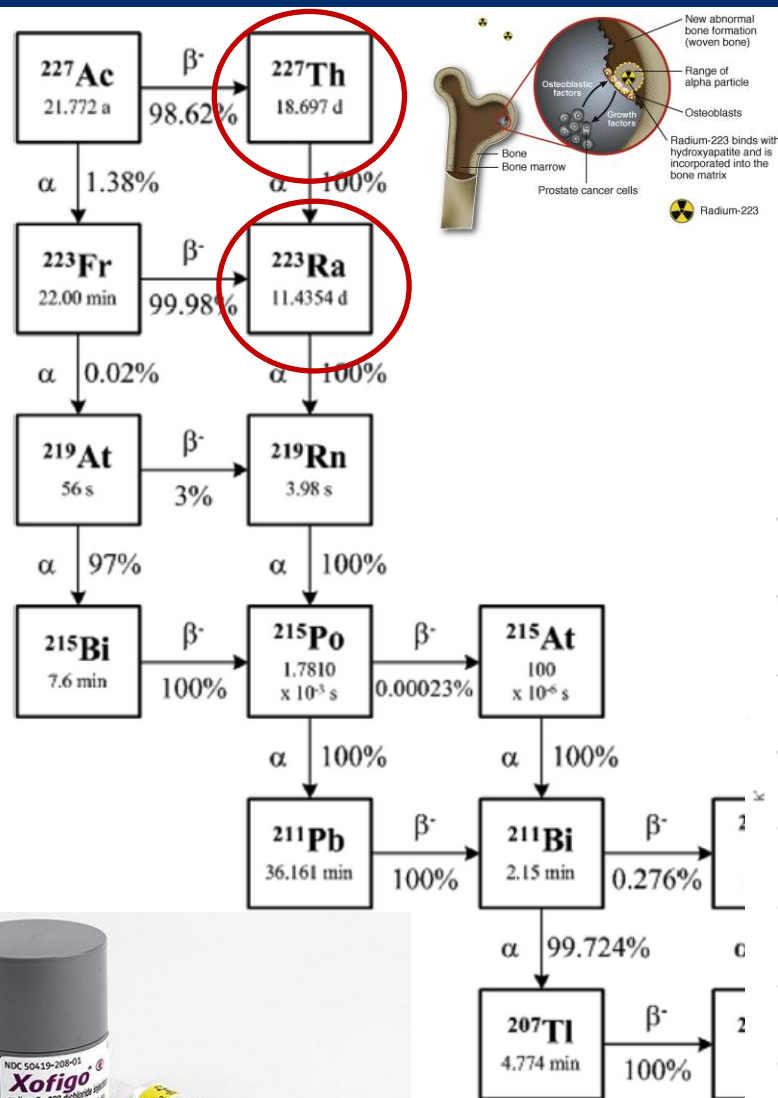
$^{139}\text{Ce}$  after  $< 0.021\%$

Load  $^{155}\text{Tb}$  and  $^{139}\text{Ce}$  in 8M  $\text{HNO}_3$ /0.1M  $\text{NaBrO}_3$

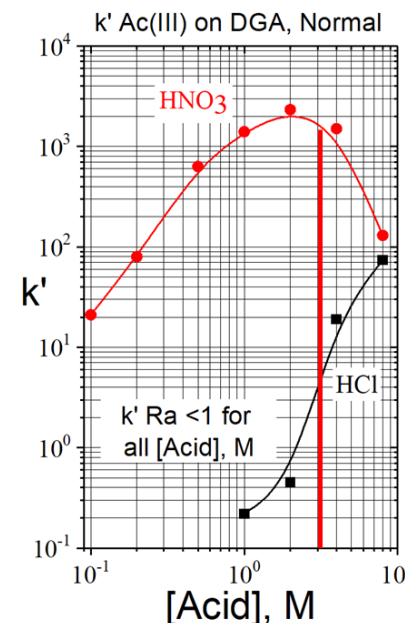


57 La 138.91 Lanthanum	58 Ce 140.12 Cerium	59 Pr 140.91 Praseodymium	60 Nd 144.24 Neodymium	61 Pm [144.91] Promethium	62 Sm 150.36 Samarium	63 Eu 151.96 Europium	64 Gd 157.25 Gadolinium	65 Tb 158.93 Terbium	66 Dy 162.50 Dysprosium
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# Purification of $^{223}\text{Ra}/^{227}\text{Th}$

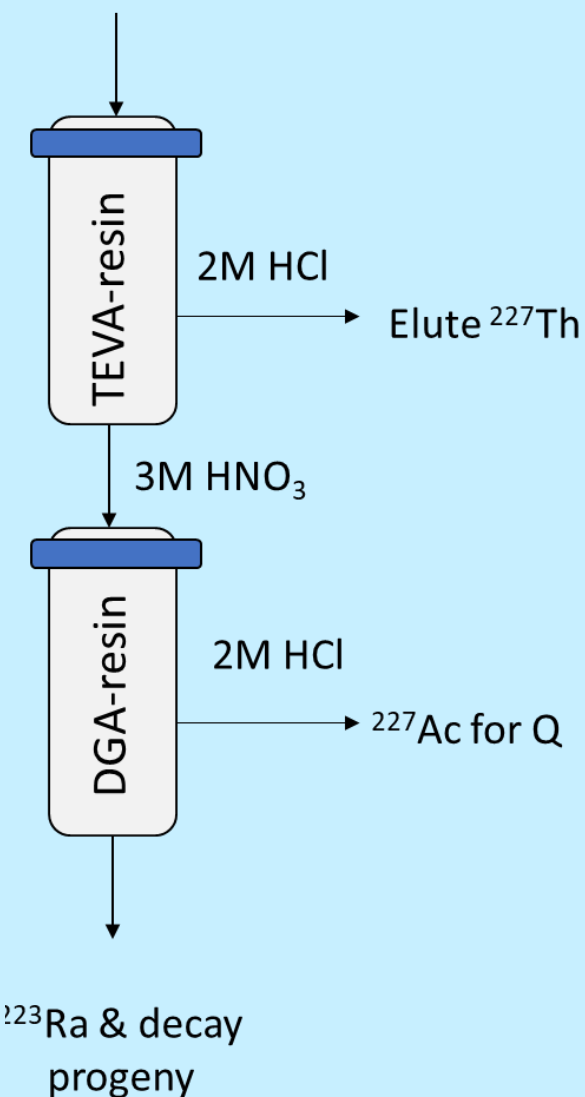


TEVA



DGA

$^{227}\text{Ac}$ ,  $^{227}\text{Th}$  and  $^{223}\text{Ra}$  in 3M  $\text{HNO}_3$





# Ac-225 Production at CERN-MEDICIS



TARGET  
PREPARATION



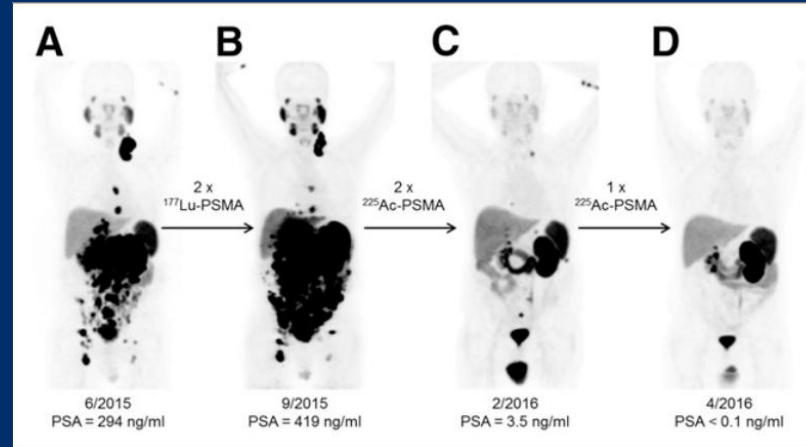
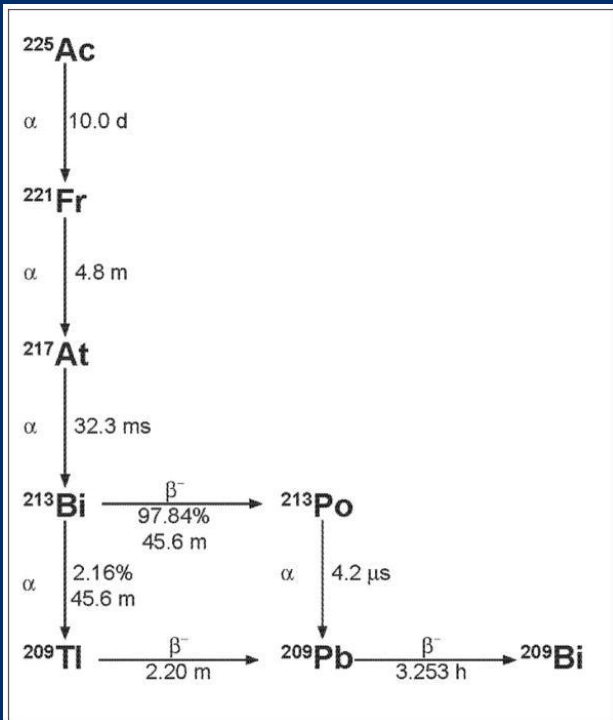
RADIONUCLIDE  
PRODUCTION



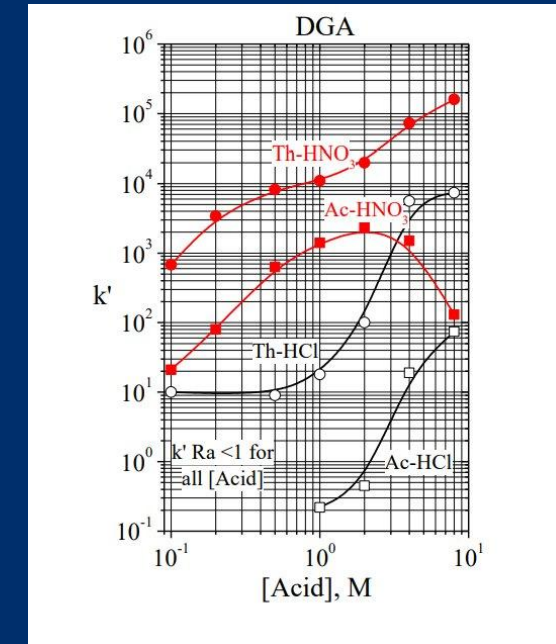
RADIONUCLIDE  
PURIFICATION

Production Facility -  
CERN - MEDICIS

Purification Scheme -  
NPL



- $^{225}\text{Ac}$  recovery of 97%
- <0.1% Ra-225 in the  $^{225}\text{Ac}$  solution



Load in 2 M  $\text{HNO}_3$

Rinse in 2 M  $\text{HNO}_3$

Elute in  $\sim 0.1$  M  $\text{HCl}$

# Ac-225 Production at Birmingham (MRIP)

Exploring novel production routes for emerging medical radionuclides in the UK.

- REAPAR: Research of the End-to-end Accelerator Production of Ac-225 from Ra-226

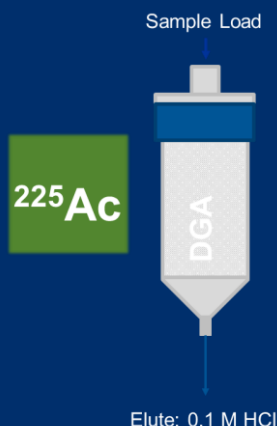


Production Facility -  
UoBirmingham

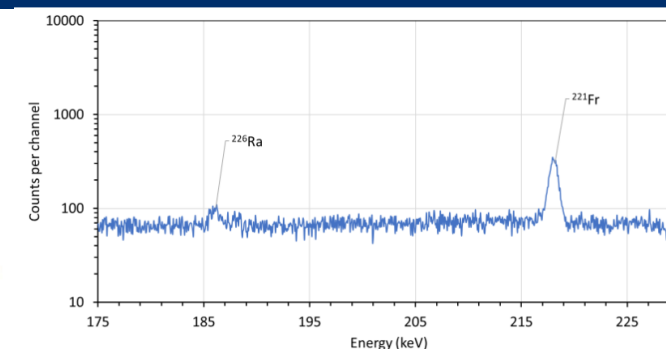
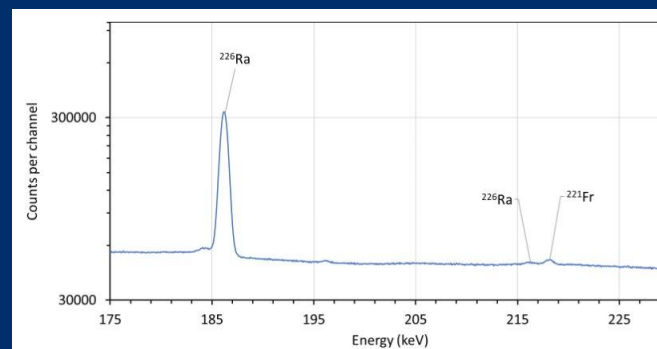


- Irradiation: 18 MeV protons, 10  $\mu\text{A}$ , 4 hours

Purification Scheme –  
NPL



Contaminant Assessment –  
NPL



$^{226}\text{Ra}$  effectively removed from  $^{225}\text{Ac}$

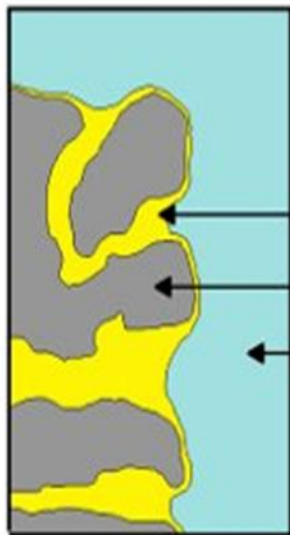
# Purification of $^{212}\text{Pb}$ from $^{224}\text{Ra}$ generator

## Secondary purification method developed:

☉ Pb-212 (Ra-224)

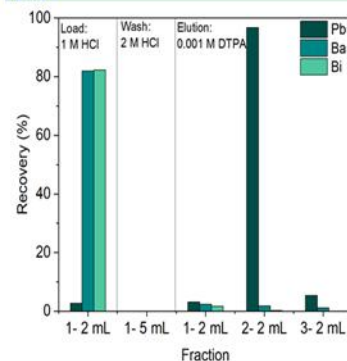


Surface of Porous Bead



Stationary Phase  
Inert Support  
Mobile Phase

### (b) 0.001 M DTPA



Total % in 4 mL eluted fraction:

Pb: 100  
Ba: 4

1 - Load  
2 mL 1 M HCl

$^{212}\text{Pb}$

TK101

SR

2- Wash  
5 mL 2 M HCl

3 - Pb-elution – 3x 2 mL fraction

- (a) 8 M HCl
- (b) 0.001 M DTPA
- (c) 0.1 M ammonium oxalate
- (d) 0.1 citric acid



- Method validation completed using a  $^{224}\text{Ra}/^{212}\text{Pb}$  generator from Oak Ridge National Laboratory, which was set-up by Barts Cancer Institute QMUL for first-in-UK  $^{212}\text{Pb}$ -labelled peptide studies.
- Post-purification, initial radionuclide calibrator measurements were completed at NPL to generate a calculator for  $^{212}\text{Pb}$  activity determination.



# Nuclear Metrology Group:

Hibaaq Mohamud, Anu Bhaisare, Frankie  
Falksohn, Alex Tribolet, Ben Russell, Seán  
Collins, Paddy Regan and Peter Ivanov



Department for  
Science, Innovation  
& Technology







UK Atomic  
Energy  
Authority

# Plasma to Patients

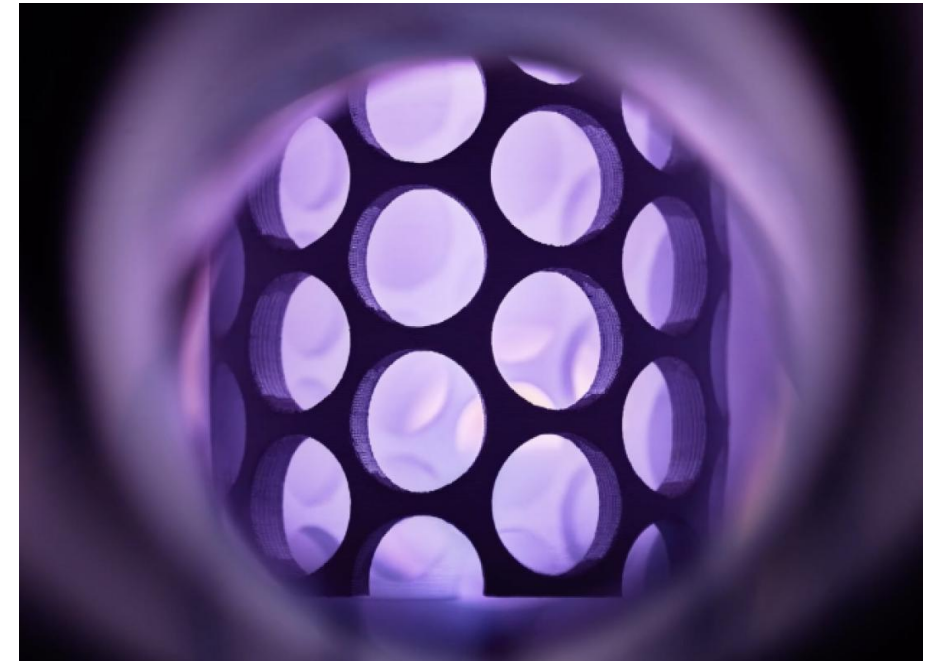
Ram Mullur, Astral Systems



Pioneering Multi State Fusion  
For Isotopes Production –  
*Plasma to Patients*

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UKAEA: Fusion for Medical  
Radionuclides;  
September 24<sup>th</sup>, 2025,  
Culham Campus, Oxfordshire, UK





# A Nuclear Medicine Renaissance is on...



- 15 Radiopharmaceuticals approved in last 10 years
  - vs. 6 in the same period prior
- Pipeline of 500 + drugs → and growing
- Increasing # of startup Radiopharma, CDMOs and Isotope companies
- Over € 22B invested over the past decade (vs. € 7B in Fusion)



NOVARTIS



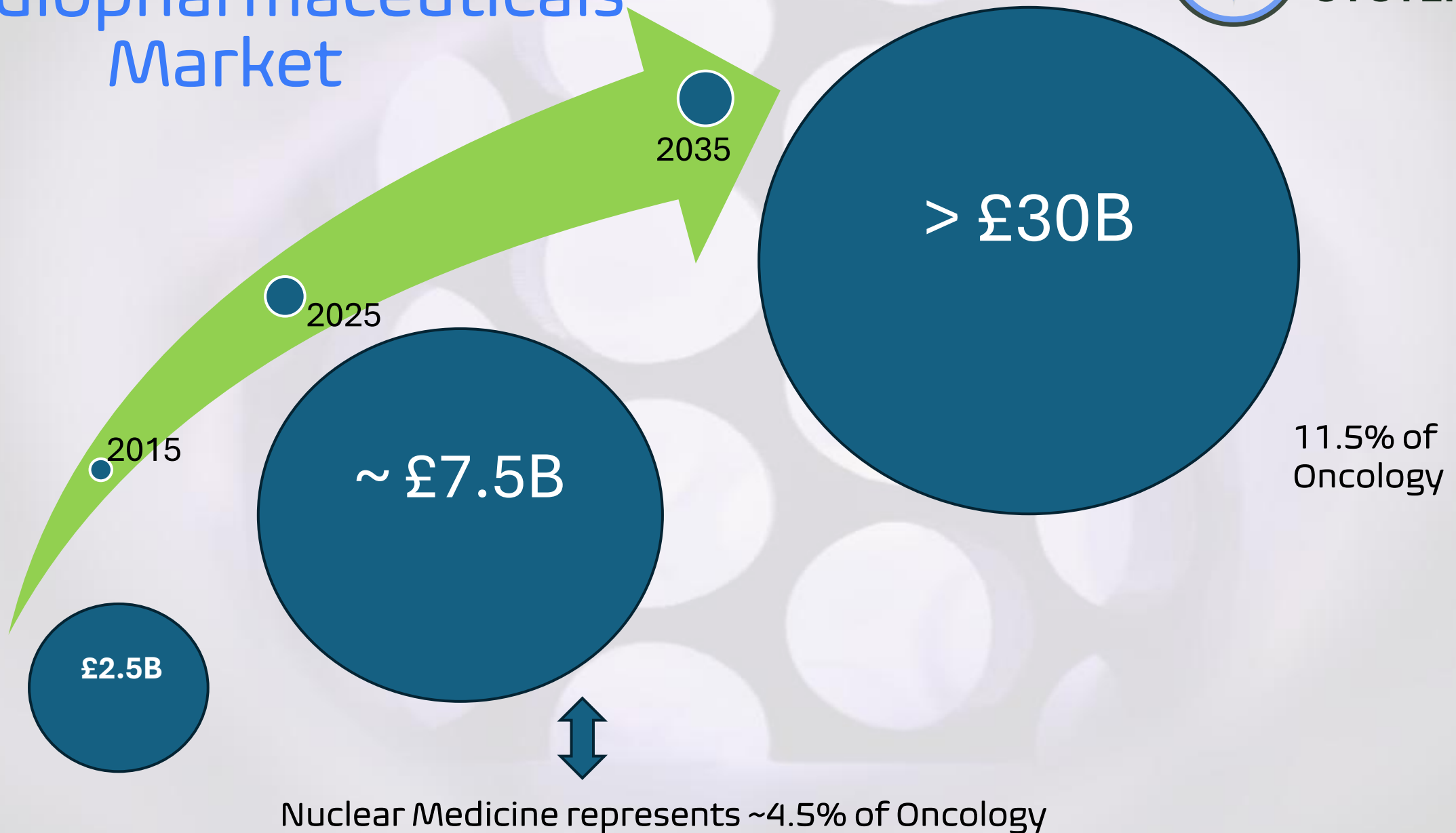
Bristol Myers Squibb™

AstraZeneca



- Radiopharmaceuticals → moving up in the Cancer-treatment hierarchy
- Novel Therapy and Theranostic radioisotopes → drug R&D pipeline: Cu-64/67; Ac-225; Pb-212....
- Quest is on for novel routes to produce isotopes
  - New Generator-based options; Power/Utility reactors, Accelerator-based platforms....
  - Astral advancing Fusion Neutrons for isotopes – Furthering UK's Leadership in Fusion

# Global Radiopharmaceuticals Market



# Trends Driving Isotopes and Radiopharmaceuticals



Advances in Imaging  
and Computing

Advent of “new”  
Beta, Alpha and  
Positron **Emitting**  
**Isotopes**

Advances in Linkers  
and Chelating  
Chemistry

Advances in Disease-  
Biomarkers → Precision  
Medicine

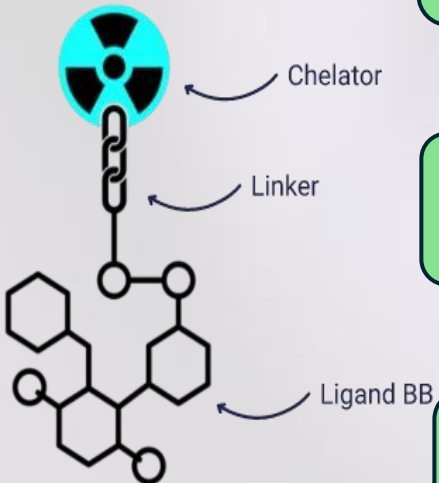
Demography and  
Epidemiology

Existing treatments  
reaching their  
limits

=

Quest for new  
tools to beat (treat)  
cancer

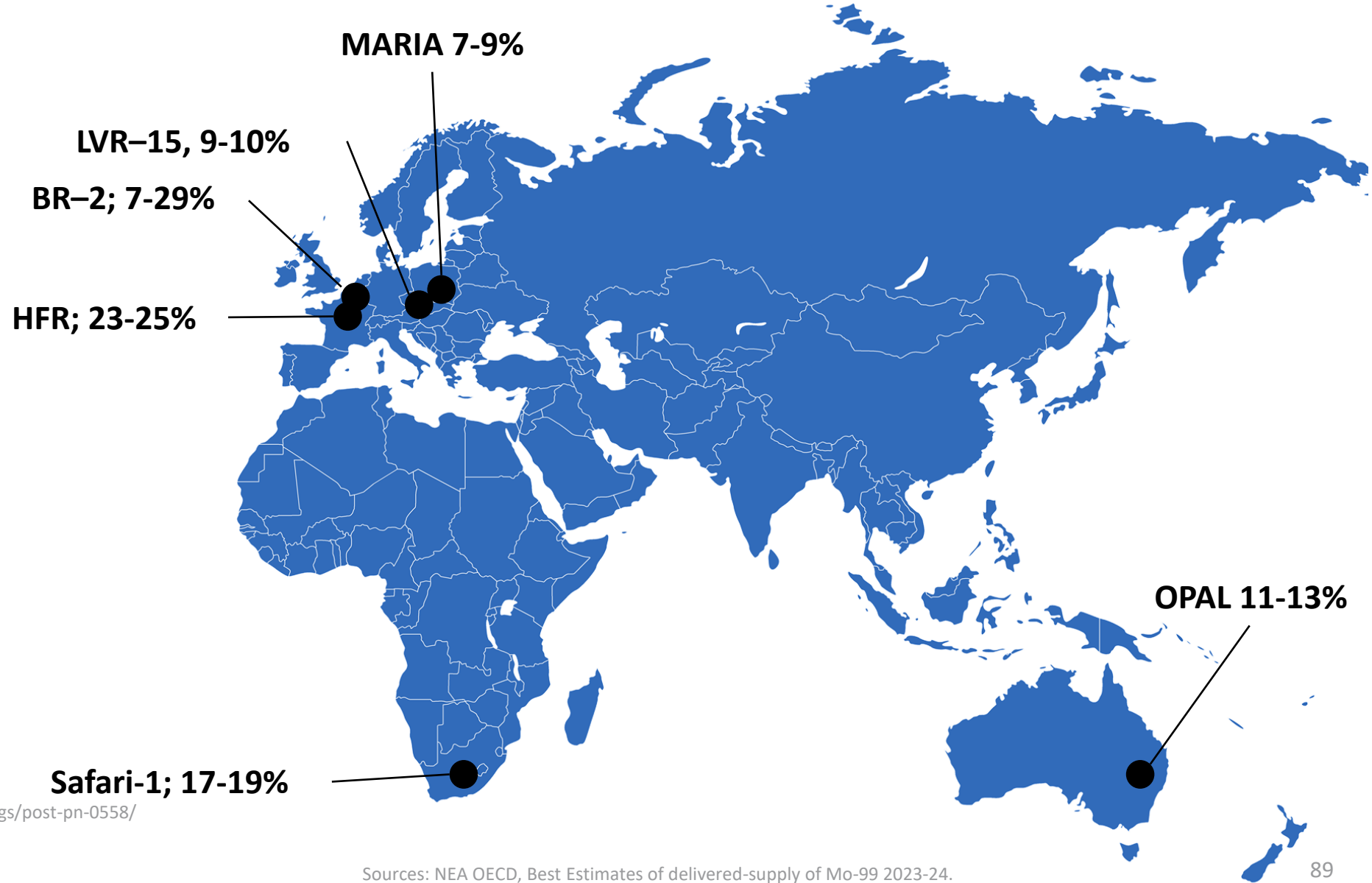
**Cancer Treatment and  
Patient Care**





# Isotope needs are rising, key infrastructure projections are dire

- Neutron Reactors - few across the world
- 6 reactors produce **85%** of reactor made medical radionuclides
- Many scheduled to slow / shut down 2030 → 2035
- 72% of global capacity will be lost within the next decade



Sources: <https://post.parliament.uk/research-briefings/post-pn-0558/>

Sources: NEA OECD, Best Estimates of delivered-supply of Mo-99 2023-24.

# UK in shortage of critical Isotopes and Radiopharmaceuticals

- **Over 750,000** radionuclide-drug procedures are carried out in UK each year
  - 100% of therapeutic isotopes and 80% of diagnostic isotopes **imported**
- Numerous medical radionuclide **shortages** in recent years
  - I-131 shortage has been especially prominent
  - Only projected to get **worse**

# UK in shortage of critical Isotopes and Radiopharmaceuticals

Sovereignty and Security of critical medical Isotopes are undeniable imperatives

Costs of not achieving them are high and the overall impact on healthcare could be deleterious



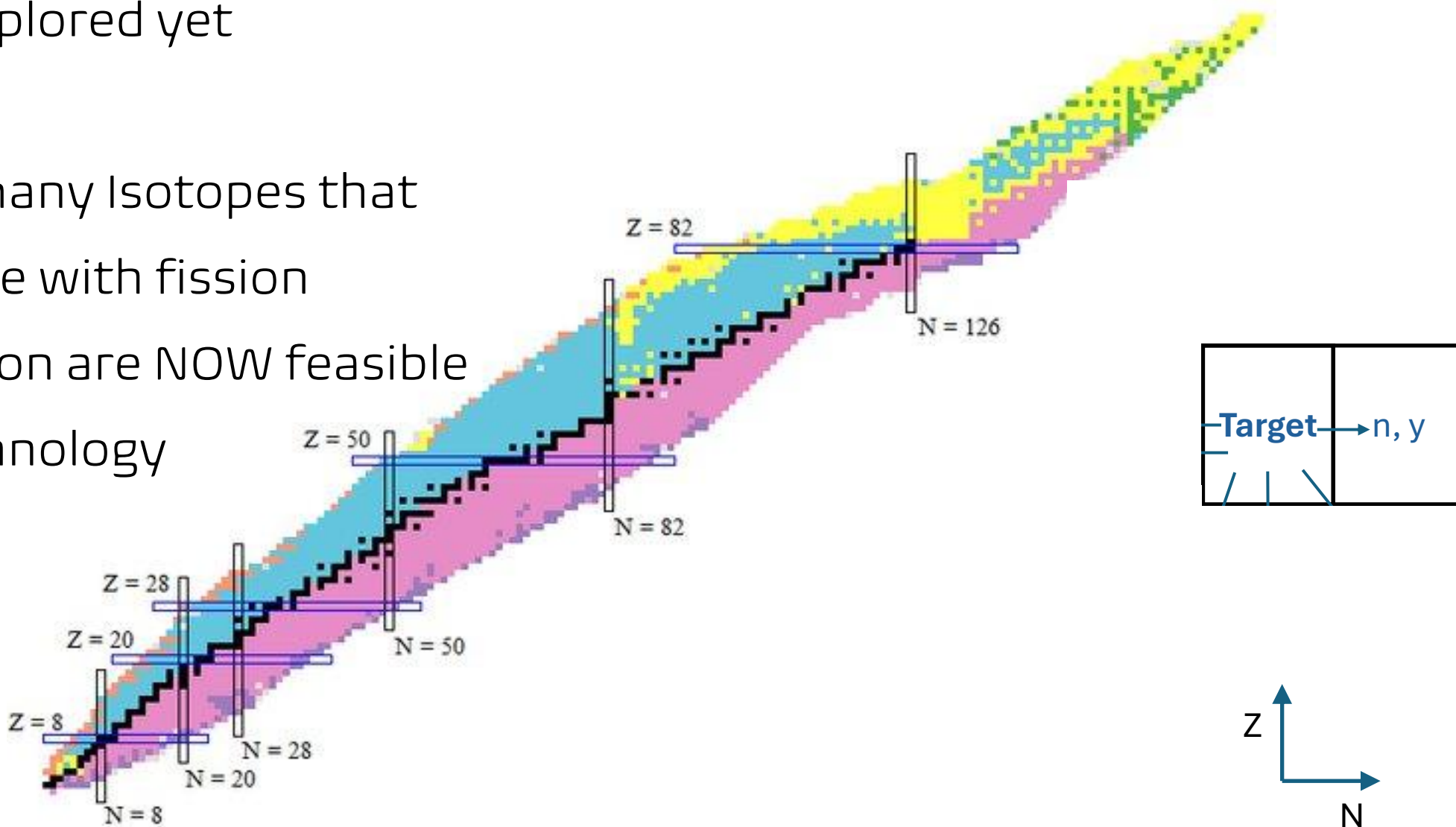
# Astral is Changing Paradigms

- Enabling an efficient and effective neutron platform for fusion research and isotope production:
  - Greater neutron output
  - Greater flux ( $\text{n/cm}^2/\text{s}$ )
  - Greater continuous fluence
- Superior economics:
  - 1000+ hours of DD neutron time logged
  - 6 commercial DT reactors in Q1, 2026
  - Low cost per neutron – highly scalable
- Will decentralise neutron-produced isotopes as cyclotrons did for proton-produced isotopes
- Will usher better prospects for isotopes production by addressing many of the existing supply vulnerabilities



# Unlocking Novel Pathways to Isotopes

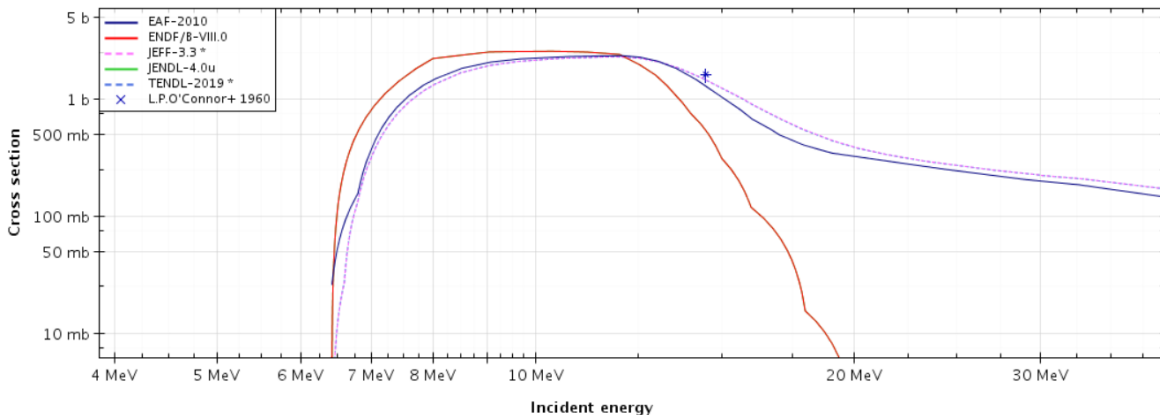
- Threshold 14MeV reactions which have not been explored yet
- Production of many Isotopes that could not be made with fission neutron-irradiation are NOW feasible with Astral's Technology



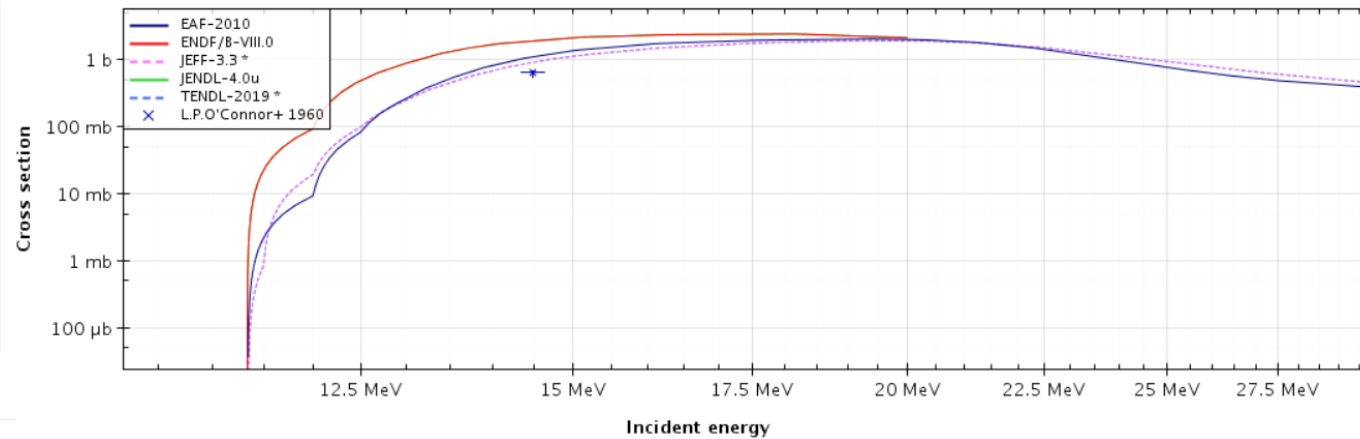
# McMaster University Collaboration

- Production of Ac-225 and Pb-212 using 14 MeV neutrons
- Ra-226 (n,2n) Ra-225  $\rightarrow$  Ac-225
- Ra-226 (n,3n) Ra-224  $\rightarrow$  Rn-220  $\rightarrow$  Po-216  $\rightarrow$  Pb-212

Ra226 (n,2n) or Ra225 production



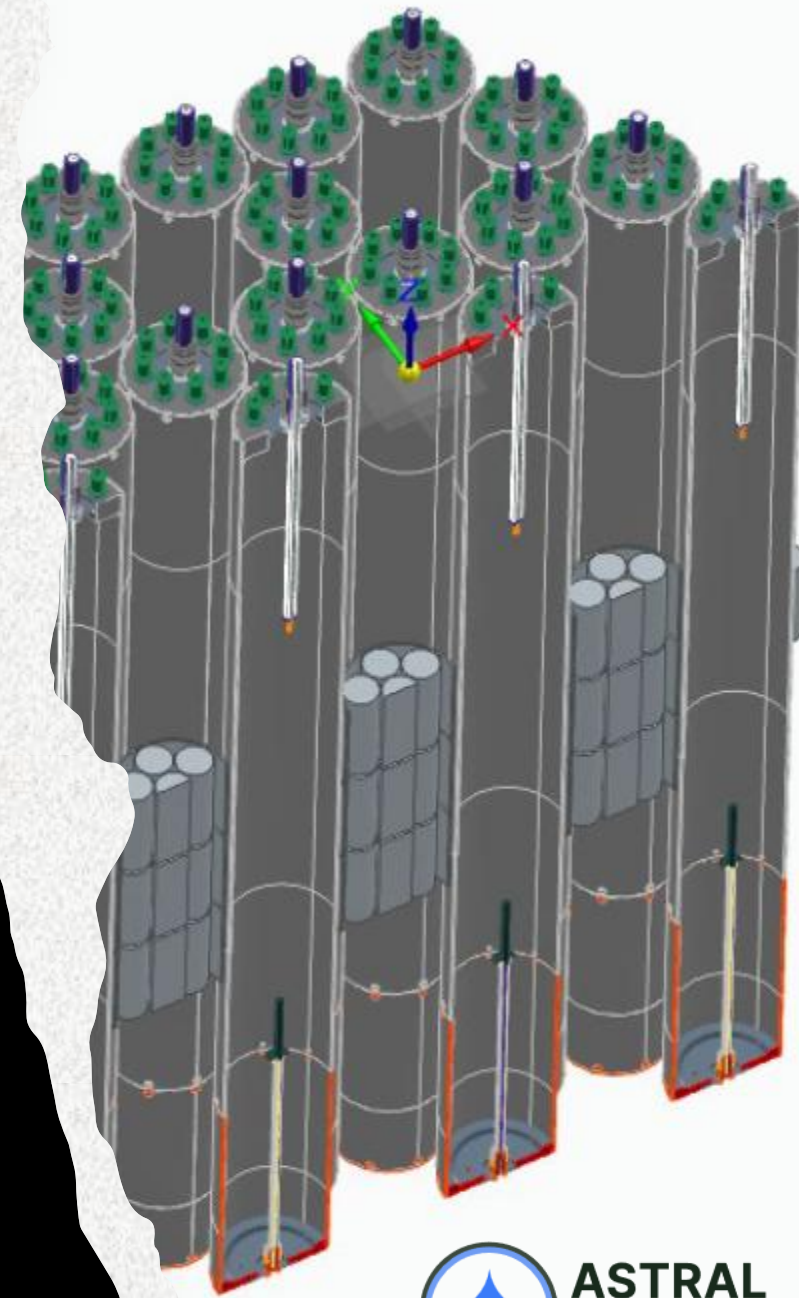
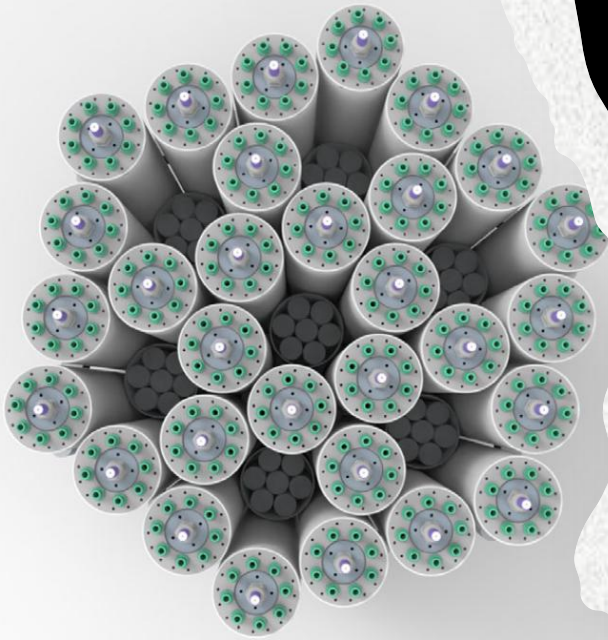
Ra226 (n,3n) or Ra224 production





# Join us to positively impact healthcare

- Compact, Modular and Cost-effective target (neutron) irradiation platform
- High Energy neutrons, higher output, fluence & flux - enabling unprecedented (n, Xn) reaction pathways (for medical isotopes)
- We invite all interested parties to propose your medical isotope production experiments and research projects
- Fusion neutrons available now!







UK Atomic  
Energy  
Authority

# Fusion Technologies for Industrial and Medical Applications

Ross Radel, SHINE



# Fusion Technologies for Industrial and Medical Applications



Ross Radel, PhD, PE

September 2025





# SHINE's Sustainable Approach To Fusion

VISION: TRANSFORMING HUMANKIND THROUGH FUSION TECHNOLOGY

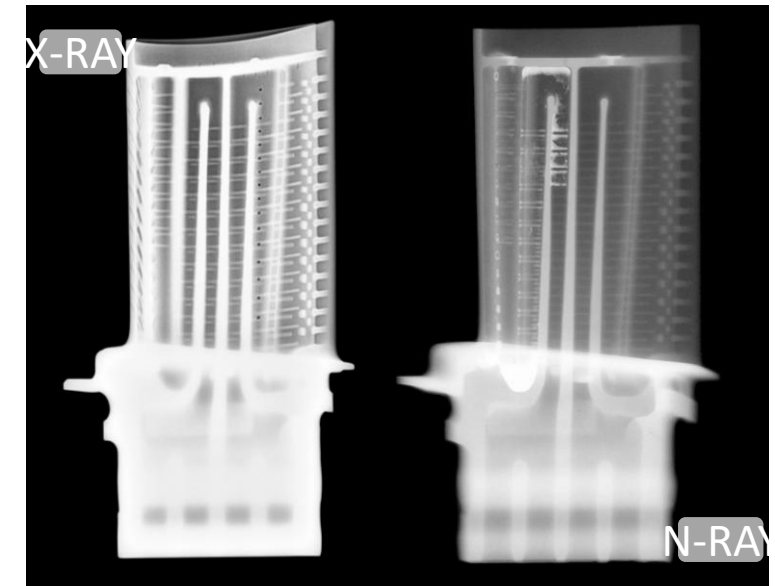
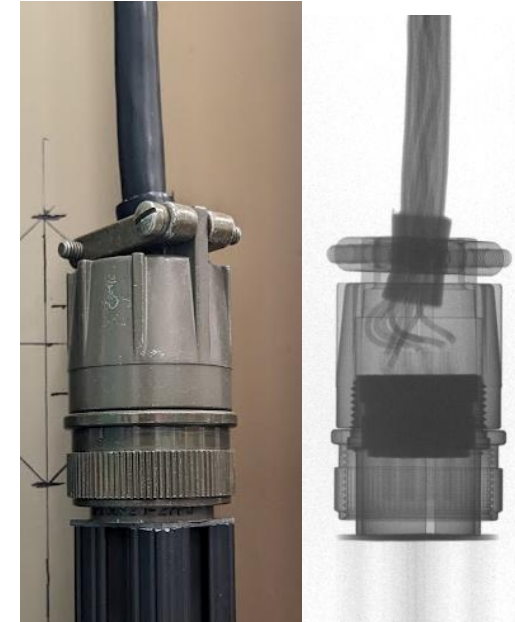
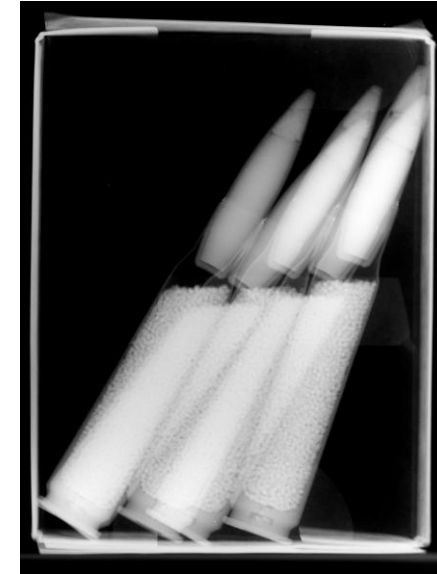
- Phase 1: Delivering low-yield fusion systems (DD & DT) for 10+ years
  - Years of licensing facilities with activation, tritium, rad waste disposal, etc.
  - Providing commercial neutron radiography and DT irradiation services
- Phase 2: Producing medical isotopes
  - Delivering Lu-177 for cancer treatment; scaling up volume quickly
  - Commissioning Mo-99 facility with 8 fusion systems, fission subcritical assemblies, and aqueous U/FP processing
  - Significant experience gained in nuclear construction, licensing, rad waste mitigation/handling, and decommissioning planning
- Phase 3: Applying lessons learned towards aqueous UNF recycling pilot facility and investigating fusion-enabled transmutation
- Phase 4: Looking for opportunities to apply our hard-gained experience and expertise to support the broader fusion community





# SHINE's Phoenix Neutron Imaging Center

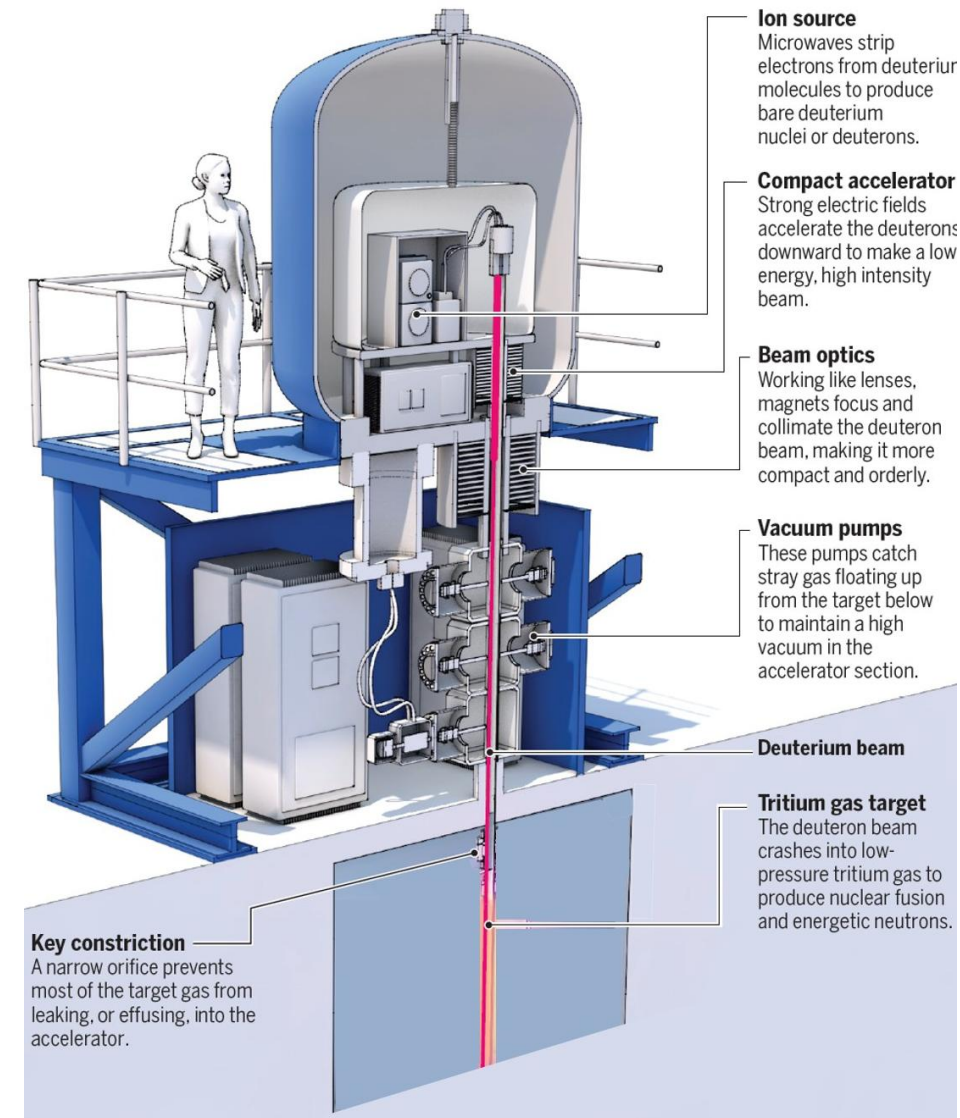
- Accelerator-based neutron source removing the reliance on aging reactor sources for neutron imaging and neutron radiation testing
  - 10,000 square foot (929 m<sup>2</sup>) facility in Fitchburg, Wisconsin, USA
- The neutron source has been in commercial operation since 2020 and has been reliably operated with >99% uptime during this period
- Built from the ground up as a neutron radiography facility
  - Thermal and fast neutron beamlines for 2D imaging, 3D CT imaging, & radiation effects testing
  - ASTM Category I Image Quality
- The facility maintains active compliance/permits with:
  - ATF, ITAR, and DoD explosives safety and security regulations
  - Aerospace & defense quality programs (ISO9001, AS9100, NAS410, etc.)



# High-Yield DT Neutron Source Overview

## Sequence of Operation

1. Microwave ion source creates dense deutron plasma
2. Up to 80 mA of deutron current extracted ( $\sim 90\%$   $D^+$ )
3. DC accelerator extracts ion beam (up to 315k V)
4. Magnetic solenoid focus and x-y steering of ion beam
5. Differential pumping system maintains target pressure at  $\sim 40$  Torr while keeping accelerator pressure under 40 mTorr
6. Beam strikes tritium (or deuterium) gas target and generates neutrons in a line source as the beam slows in the gas
7. Up to  $4.6 \times 10^{13}$  n/s measured output

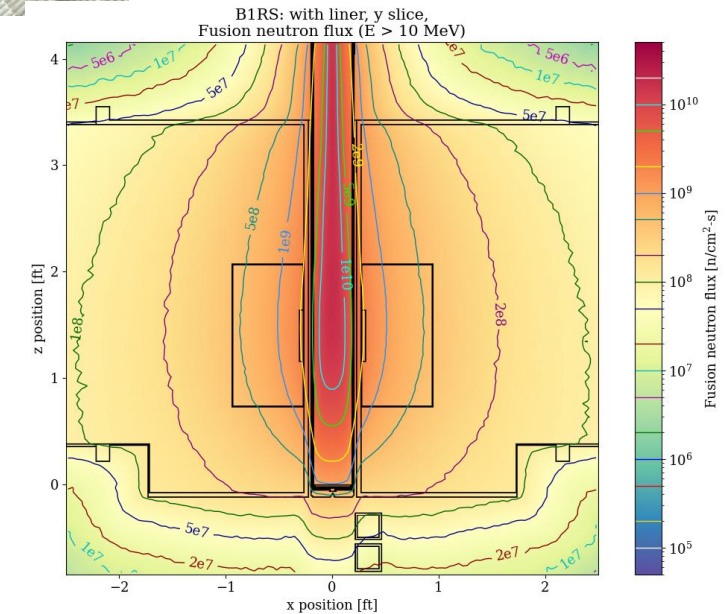
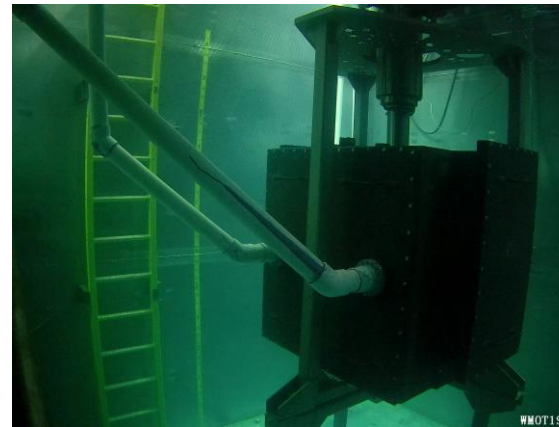




# FLARE™ by SHINE – Rad Hard Testing

## FUSION LINEAR ACCELERATOR FOR RADIATION EFFECTS

- SHINE launched a high flux 14 MeV neutron irradiation service in 2023
  - $>1.3 \times 10^{13}$  n/s DT steady-state source
  - $4.1 \times 10^9$  n/cm<sup>2</sup>/s for small irradiation volumes
- Supports multiple test protocols including single event effects, displacement damage, and total ionizing dose
- Very large irradiation cavity supports testing of multiple parts and conditions simultaneously
- Fully licensed facility for tritium handling and DT neutron operations



Available fast flux with  $1.3 \times 10^{13}$  n/s source

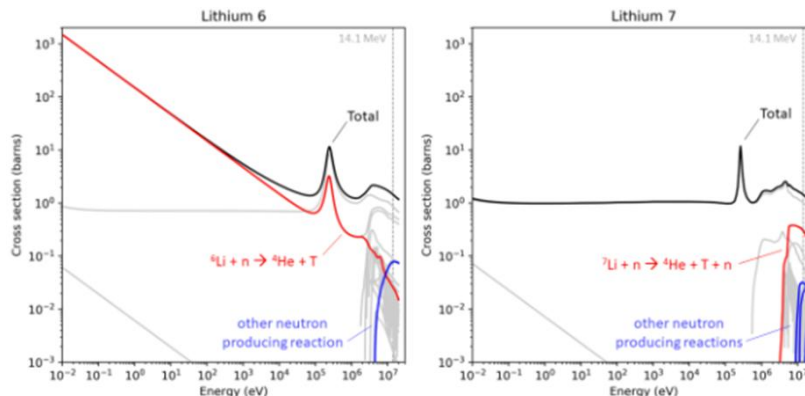
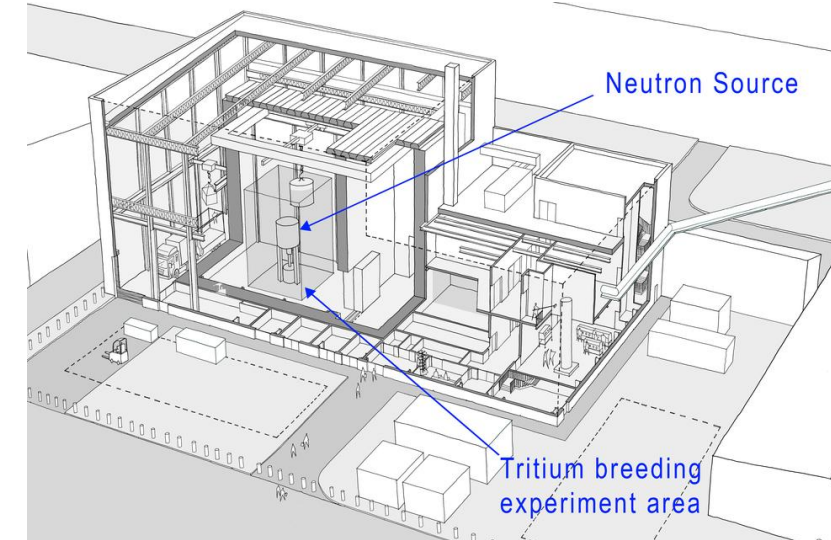
*Service commercially available to fusion system developers,  
defense microelectronics survivability community, and others*



# Lithium Breeding Tritium Innovation (LIBRTI)

## UKAEA PROGRAM FOR ENGINEERING-SCALE BREEDING BLANKET TESTING

- Tritium breeding is needed for a sustainable fusion fuel cycle
- SHINE was selected to provide a FLARE-like DT neutron source to support the UKAEA LIBRTI project
- Breeder blanket efficacy is critically dependent upon both neutron and tritium transport
  - Most tritium is created ~1m from the source as neutrons need to be moderated to where the X-section is more potent
  - Uncertainty increases as you move through the blanket
  - High starting flux needed for neutrons to reach that deep
- LIBRTI is the first facility to construct a breeder test on this scale and with **sufficient neutron flux / T<sub>2</sub> production**

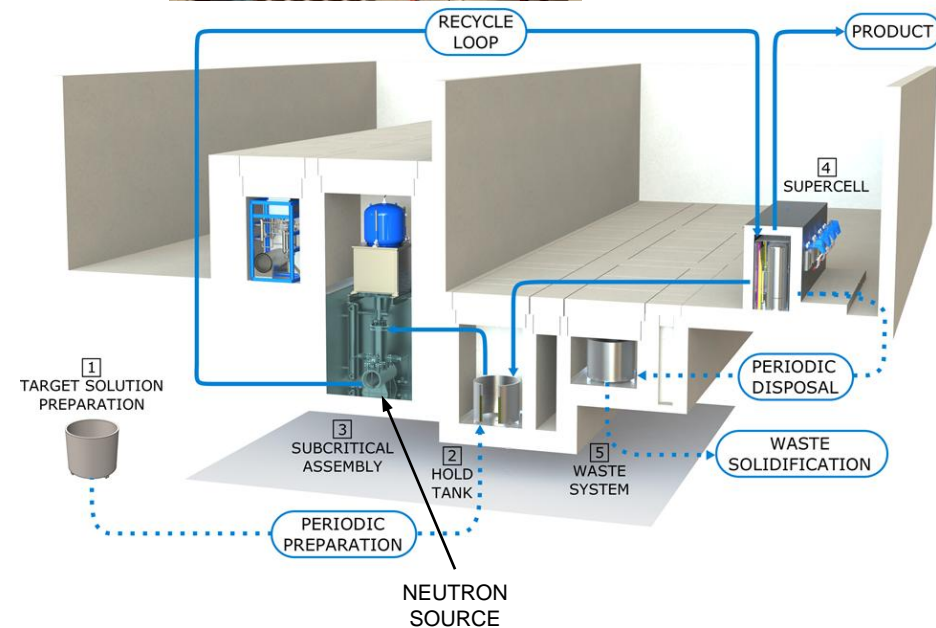
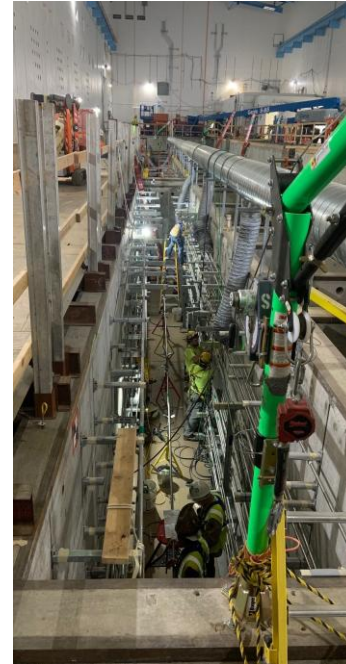


Neutron Source



# Fission Product Production/Separation

- Licensed and constructed. Installing process equipment.
- Facility will produce >50% of US **Mo-99** demand
  - 10 CFR Part 50 SER issued in early 2023
  - 2<sup>nd</sup> site in Netherlands; site eval and licensing underway
- Production Process
  1. Irradiate aqueous LEU subcritical assembly with DT neutrons
  2. Up to 1 MW of fissions produced
    - a. Mo-99 has a ~6% cumulative yield
  3. Transfer  $\text{UO}_2\text{SO}_4$  to hold tanks and then the “super cell”
    - a. Mo-99 extraction, purification, QC & packaging
  4. Final products ship to customers





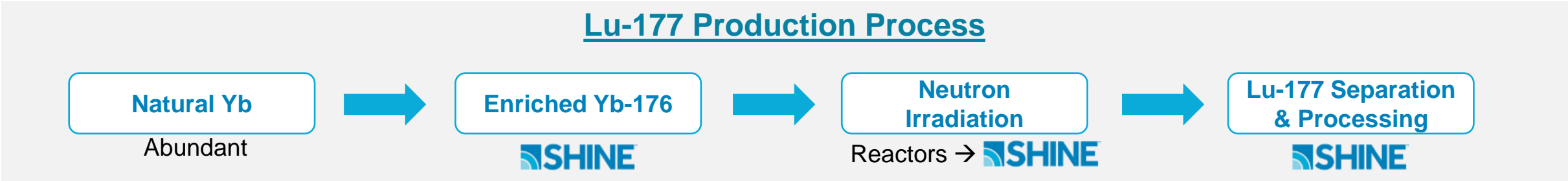
# Therapeutic Isotopes

- SHINE has established the largest source of GMP Lu-177 currently produced in North America
  - Emergent cancer treatment isotope – rapidly growing usage
- Vertical integration & technology are our key differentiators
  - Yb Enrichment: We produce our own Yb-176 via in-house electromagnetic separation
  - Irradiation: Research reactors today; moving to in-house irradiation
  - Lu-177 Processing: Our technology provides significant scaling advantage
- Producing GMP Lu-177 from our Wisconsin, USA facility
  - 200,000 doses per year capability

Lu-177 Product Spec Sheet

PACKAGING OPTIONS	10 mL flat bottom vial 2mL conical glass vial
CHEMICAL FORM	n.c.a. <sup>177</sup> LuCl <sub>3</sub> in 0.04M HCl solution
SPECIFIC ACTIVITY	≥3,000 GBq/mg at SHINE calibration time
RADIOCHEMICAL PURITY	≥99% as <sup>177</sup> LuCl <sub>3</sub>
RADIONUCLIDIC PURITY	≥99.9% <sup>177</sup> Lu
RADIOACTIVITY CONCENTRATION AT SHINE CALIBRATION TIME	1.0Ci/mL (37GBq/mL) +/-10%
EXPIRY	10 days from the end of synthesis
STANDARD CALIBRATION TIME	Tuesdays 1200 central time (U.S) at 1.00 Ci/mL
CGMP	Meets ICH-Q7 and FDA-21CFR
RADIOLABELING YIELD	≥ 99%

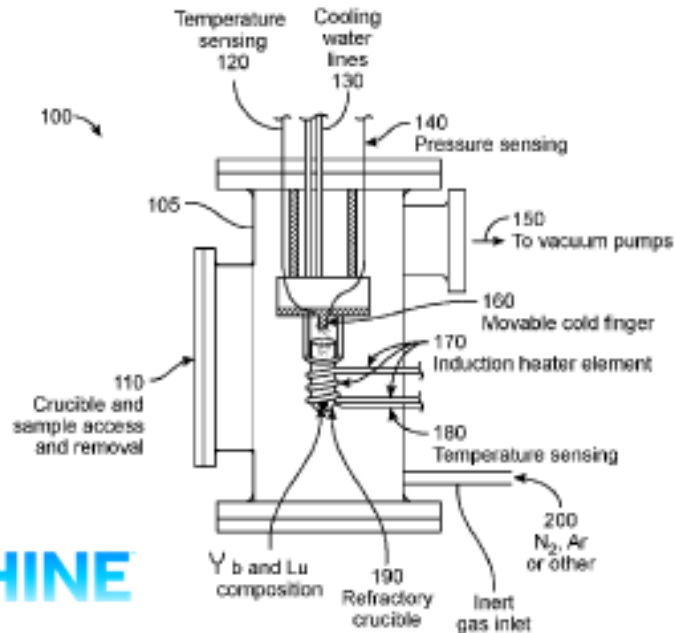
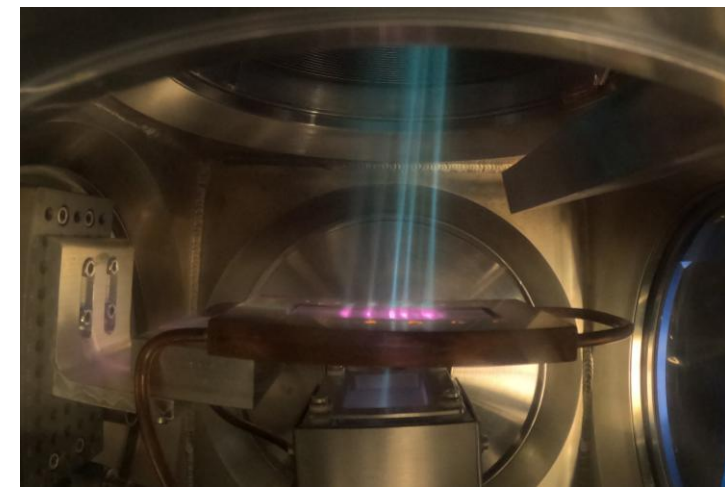
## Lu-177 Production Process



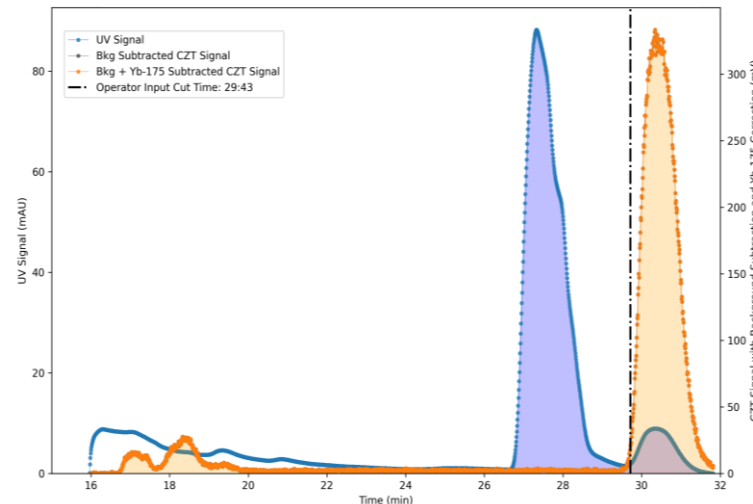


# Enrichment, Separation, and Purification

- Built/operating enrichment system with improved yield/purity
  - 50 g/year Yb-176 production per machine; >99.7% isotopic purity
  - Producing more Yb-176 than is being consumed today
- Thermal distillation separation relies on the wide variation of boiling points (Lu: 3,402 °C, Yb: 1196 °C)
  - Irradiated metal target is heated under vacuum
  - Yb sublimates and is captured on a cold trap; Lu concentrates
- Material dissolved in HCl and purified via HPLC process
- Testing these processes for Tb-161 production today



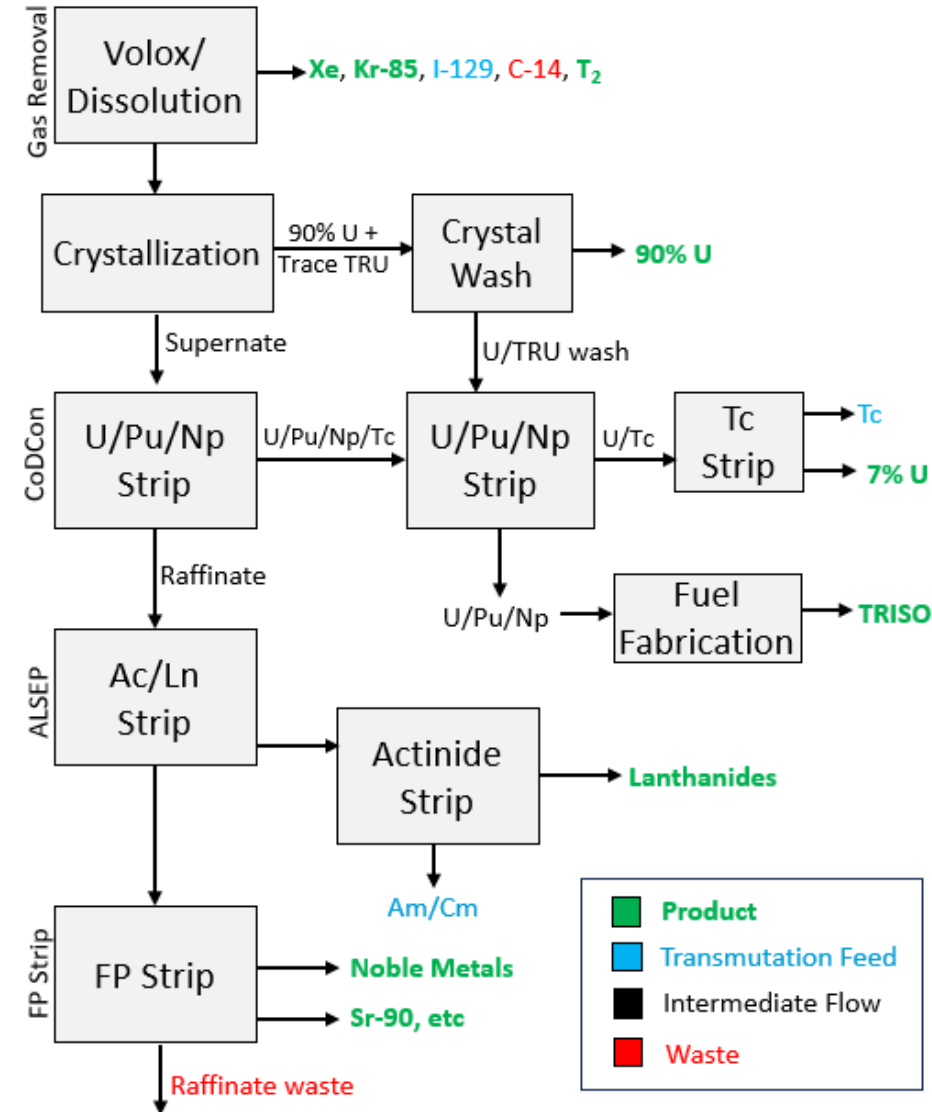
Typical HPLC Output



# UNF Recycling and Isotope Recovery

NEWLY DEVELOPED TECHNOLOGIES PRESENT UNIQUE OPPORTUNITIES

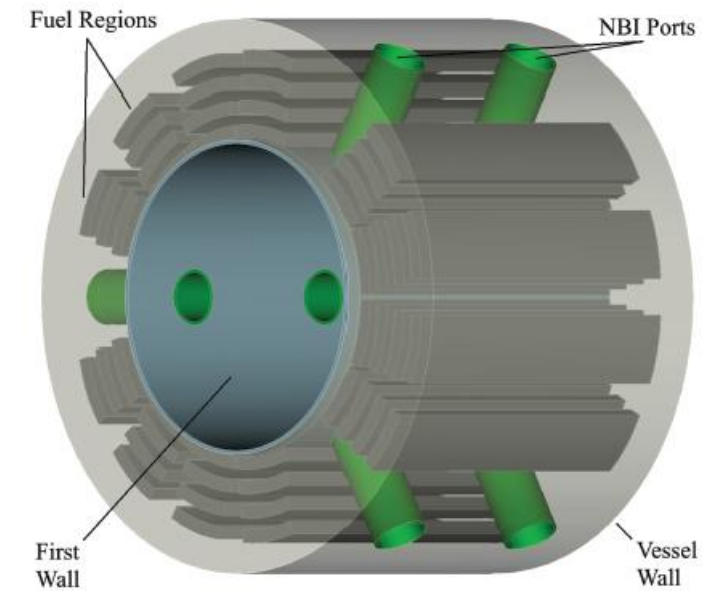
- Evaluating technology improvements for regulatory compliance and improved economics
  1. Voloxidation for pre-processing fuel assemblies allows isolation of iodine/tritium to avoid liquid contamination
  2. CoDCon extraction to create a **blended U/Pu** stream
  3. Removal of minor actinides (via ALSEP) from the waste stream; future transmutation
  4. Harvest platinum group metals and rare earth elements for sale
    - a. Am-241, Kr-85, and Sr-90 are commercial targets as well
  5. >95% of waste volume expected to be suitable for near-surface disposal
    - a. Assessing deep bore disposal with Deep Isolation
- Approach combines proven and novel technologies and implements SHINE's improved safety and regulatory approach to greatly reduce the lifecycle cost of a recycling facility



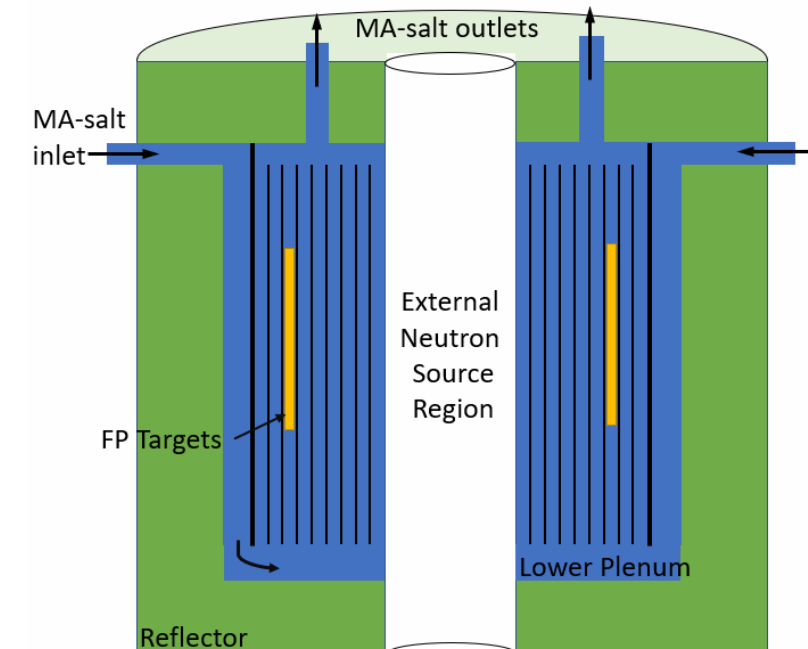
# Longer Term: Transmutation

## MAXIMIZING BENEFITS OF A CLOSED FUEL CYCLE

- Goal: Eliminate minor actinides (Np, Am, Cm) via fast fission
  - Assessing liquid (molten salt) core options
  - Evaluating fusion-driven subcritical systems and critical systems
  - Additionally, fission products (e.g. I-129 and Tc-99) can be converted to shorter lived or stable isotopes via neutron capture
- Benefits:
  - Molten cores: Minor actinides don't enter fuel rod supply chain
  - Systems can operate nearer to (or at) critical levels due to increased negative reactivity
  - Molten salt core allows for rapid capture of short-lived fission products for secondary sale
    - Significant separation development to be performed
- SHINE has been awarded an ARPA-E NEWTON grant to investigate/test blanket technologies over the next 3 years



Courtesy of UW-Madison\*







[RossRadel@SHINEfusion.com](mailto:RossRadel@SHINEfusion.com)

# **Thank you for listening!**

If you have any questions, please email  
**[innovation@ukaea.uk](mailto:innovation@ukaea.uk)**

# Agenda

## 10:30 – 12:30: Presentations

Lee Evitts & Jessica Hollis, <b>UKAEA</b>	Exploration of Fusion technology for Medical Radionuclides at UKAEA
Jennifer Young, <b>Barts Cancer Institute - Queen Mary University of London</b>	Radionuclides for Health UK

## Coffee Break

Peter Ivanov, <b>National Physical Laboratory</b>	Development of Radiochemical Purification Methods for Emerging Medical Radionuclide Standards
Ram Mullur, <b>Astral Systems</b>	Plasma to Patients
Ross Radel, <b>SHINE</b>	Fusion Technologies for Industrial and Medical Applications

## 12:30 – 13:30: Networking Lunch

## 13:30 – 14:15: Panel Discussion

Jamie Townes (UKIFS)  
Talmon Firestone (Astral Systems)  
Marta Barrabino (Tokamak Energy)  
Jennifer Young (Barts Cancer Institute - Queen Mary University of London)  
Kathy Chan - (Institute of Cancer Research)

## 14:15 – 14:30 | Closing Remarks

## 14:30 – 15:30 | Networking & Poster Session